

## EAST Steady-state Long Pulse H-mode with Core-edge Integration for CFETR

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Recent EAST experiment has successfully demonstrated long pulse steady-state high plasma performance scenario with core-edge integration since the last IAEA in 2018 [1]. A discharge with a duration over 60s with  $\beta_P \sim 2.0$ ,  $\beta_N \sim 1.6$ ,  $H_{98y2} \sim 1.3$  and internal transport barrier on electron temperature channel is obtained with multi-RF power heating and current drive, i.e.  $\sim 2.5$  MW LH2 and 0.9 MW ECH, where the plasma configuration is the upper single null with the strike points on the tungsten divertor (shown in figure 1). Loop voltage was well controlled to be zero which indicates the fully non-inductive current drive condition. Small ELMs (frequency  $\sim 100$ -200Hz) were obtained in this long pulse H-mode discharge. In the operation, the optimization of X-point, the outer gap and local gas puffing near LH2 antenna were investigated to maintain RF power coupling and to avoid formation of hot spot on the 4.6 GHz LH2 antenna. Global parameters of toroidal field  $B_T$  and line averaged electron density  $\langle n_e \rangle$  were optimized for high current drive efficiency of LH2 and for on-axis deposition of ECH. The on-axis ECH was applied not only for the core electron heating, but also for the control of high  $Z_{eff}$  impurities in the core plasmas.

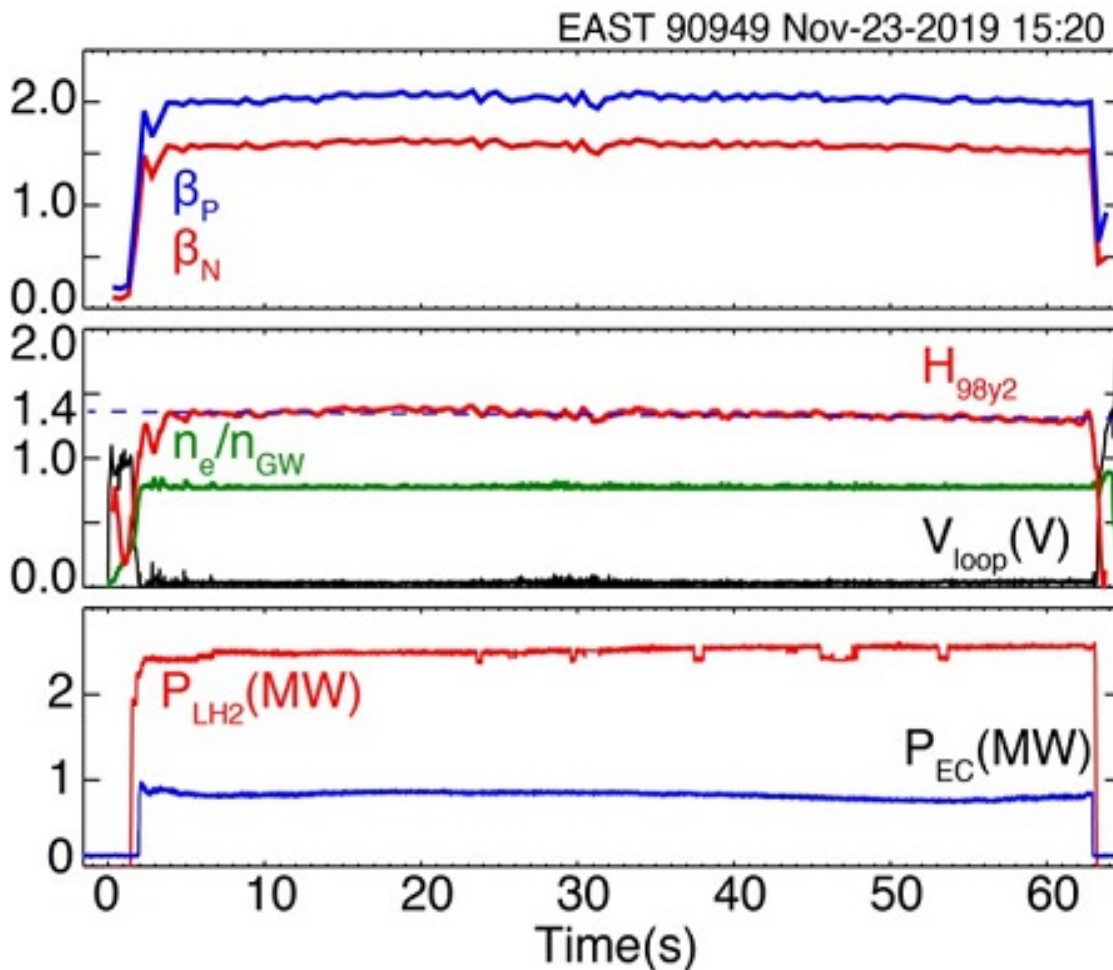


Figure 1: Time histories of several parameters for EAST 60s high  $\beta_P$  long pulse discharge.

Meanwhile, a higher  $\beta_N \sim 1.8$  with a duration of 20s is achieved by using the modulated neutral beam. Several normalized parameters of  $\beta_P$  ( $\beta_P \sim 2.0$ ),  $\beta_N$  ( $\beta_N \sim 1.8$ ),  $H_{98y2}$  ( $\sim 1.3$ ),  $n_e/n_{GW}$  ( $\sim 0.75$ ) are close or even higher

than the phase III 1GW scenario of CFETR steady-state [2]. Other features such as metal wall (tungsten divertor), low torque injection ( $\Gamma_{inj} \sim 1.0 \text{ Nm}$ ), electron dominated heating ( $T_e > T_i$ ), moderate bootstrap current fraction ( $f_{bs} \sim 50\%$ ), broaden current density profile with the central  $q(0) > 1.0$  and good energy confinement, have also been demonstrated in this scenario. Note that high-Z impurity accumulation in the plasma core was well controlled in a low level by using the on-axis ECH and reducing the fast ion losses through beam energy optimization.

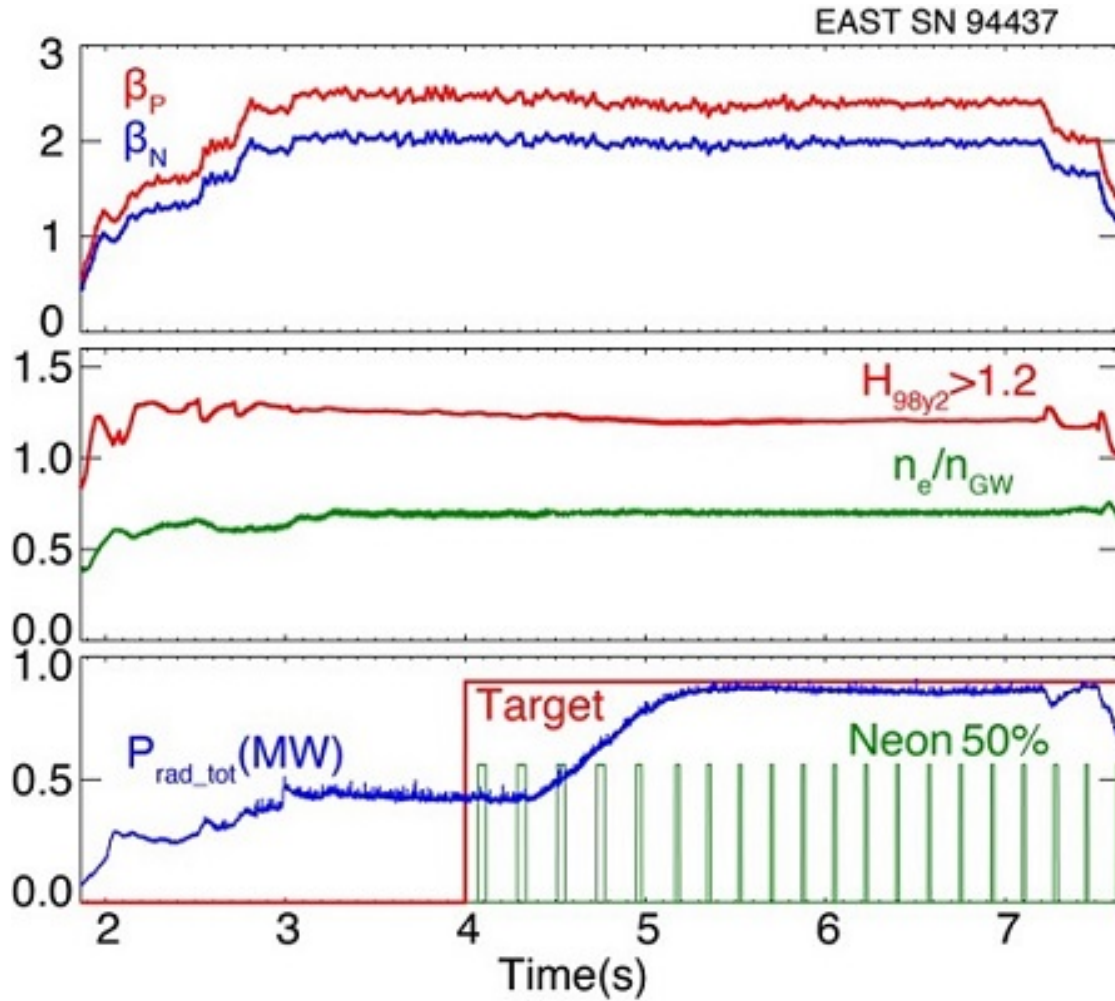


Figure 2: EAST high  $\beta_P$  discharge with core-edge integration.

More recently, EAST has demonstrated a compatible core and edge integration in high  $\beta_P$  scenarios: high confinement  $H_{98y2} > 1.2$  with high  $\beta_P \sim 2.5/\beta_N \sim 2.0$ ,  $f_{bs} \sim 50\%$  is sustained with reduced heat flux by active divertor heat flux at high density  $n_e/n_{GW} \sim 0.7$  and moderate  $q_{95} \sim 6.7$  (shown in figure 2). The energy confinement quality was almost maintained with  $H_{98y2} > 1.2$  during the radiation feedback control. By active impurity seeding through radiative divertor feedback control via radiated power, the peak heat flux is reduced by  $\sim 30\%$  on the ITER-like tungsten divertor, here a mixture of 50% neon and 50%  $D_2$  is applied. Note that EAST has developed a number of heat flux control techniques to reduce heat load in separate experiments.

In summary, recent EAST experiments demonstrated long pulse steady-state high plasma performance scenarios and heat flux feedback control. Detailed physics basis to investigate stability and particle transport will be presented for the understanding of fully integrated core-edge solutions on EAST. As a test bed for ITER and CFETR, the EAST upcoming experiments will further exploit additional heating power and demonstrate the core-edge integration of steady-state long pulse high performance scenarios with full metal walls.

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[1] X. Gong *et al* 2019 *Nucl. Fusion* **59** 086030

[2] J. Huang *et al* 2020 *Plasma Phys. Control. Fusion* **62** 014019

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