



# Development of the 435-B(U) Packaging for Radioactive Source Recoveries in International Locations

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- The Off-Site Source Recovery Program (OSRP)
- History and Background of 435-B
  - Design and Development
  - Testing
  - Manufacturing
  - Operations



# OSRP Mission and Accomplishments

**Mission: Recover excess, unwanted, abandoned, and orphaned radioactive sealed sources in the interest of global security and public health/safety**

- OSRP has contributed to national and global security by removing more than 43,000 radioactive sources, totaling over 1.3 million curies (**49,776 TBq**) of material.
- OSRP has removed sources from 27 countries worldwide.

Isotope	Sources Recovered	TBq Recovered
$^{60}\text{Co}$	6,741	12,614
$^{90}\text{Sr}$	303	23,701
$^{137}\text{Cs}$	5,397	12,163
$^{238}\text{Pu}$	2,528	587
$^{239}\text{Pu}$	1,185	49
$^{241}\text{Am}$	24,551	648
All Others	2,988	14
<b>TOTALS</b>	<b>43,693</b>	<b>49,776</b>

***OSRP is sponsored by the NNSA Office of Radiological Security (ORS), NA-212***



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# Cat 1 and 2 Devices Commonly Recovered by OSRP

Self-shielded irradiators, primarily Cs-137 and Co-60, 4.81 TBq to 148 TBq decayed

## High-Activity Beta/Gamma Devices

**Gammacell 1000**



*Isotope:*  
Cs137

*Max. Activity:*  
120 TBq

*Weight:*  
1,361 kg

**Gammacell 3000**



*Isotope:*  
Cs137

*Max. Activity:*  
120 TBq

*Weight:*  
1,588 kg

**IBL 437c**



*Isotope:*  
Cs137

*Max. Activity:*  
208 TBq

*Weight:*  
2,018 kg

**Gammacell 40**



*Isotope:*  
Cs137

*Max. Activity:*  
155 TBq

*Weight:*  
3,175 lbs

**Gammacell 200/220**



*Isotope:*  
Co60

*Max. Activity:*  
977 TBq

*Weight:*  
3,742 kg

**J.L. Shepherd 143**



*Isotope:*  
Cs137

*Max. Activity:*  
122 TBq

*Weight:*  
907 kg

**J.L. Shepherd Mark 1**



*Isotope:*  
Cs137

*Max. Activity:*  
833 TBq

*Weight:*  
1,361 kg

**Theratron 780**



*Isotope:*  
Co60

*Max. Activity:*  
496 TBq

*Weight:*  
2,495 kg

# Type B Container Development

- In 2004, the U.S. NRC revised regulations to harmonize with the IAEA's 1996 edition of "Regulations for the Safe Transport of Radioactive Material" (IAEA Safety Standards Series No. TS-R-1).
- As a result, a number of Type B packaging models used by OSRP and industry for Type B shipments were phased out of use in October 2008.
- This limited the number available Type B containers and increased the costs of Type B shipments while demand was increasing.



The 20-WC was a Type B packaging in common use prior to the regulation change

# Type B Container Development

- NNSA's Office of Radiological Security directed OSRP to design, test, certify and fabricate new Type B container models in anticipation of upcoming shortages of compliant containers.
- NNSA and OSRP began work on two type B containers in 2009.
  - The 435-B, a smaller, non-shielded packaging.
  - The 380-B, a large, shielded packaging.
- OSRP worked with federal entities, industry experts, the IAEA and a other stakeholders over the course of a decade to bring the new models into operation.

# Design and Development of 435-B



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# 435-B Development History

2009: Project Funding Approved

2011: Prototype Drop Testing

2013: SAR submitted to USNRC

2014: Design approved by USNRC

2016: First two units fabricated

2018: First use of the 435-B

2019: Third unit donated to IAEA

2019: Fourth unit fabricated



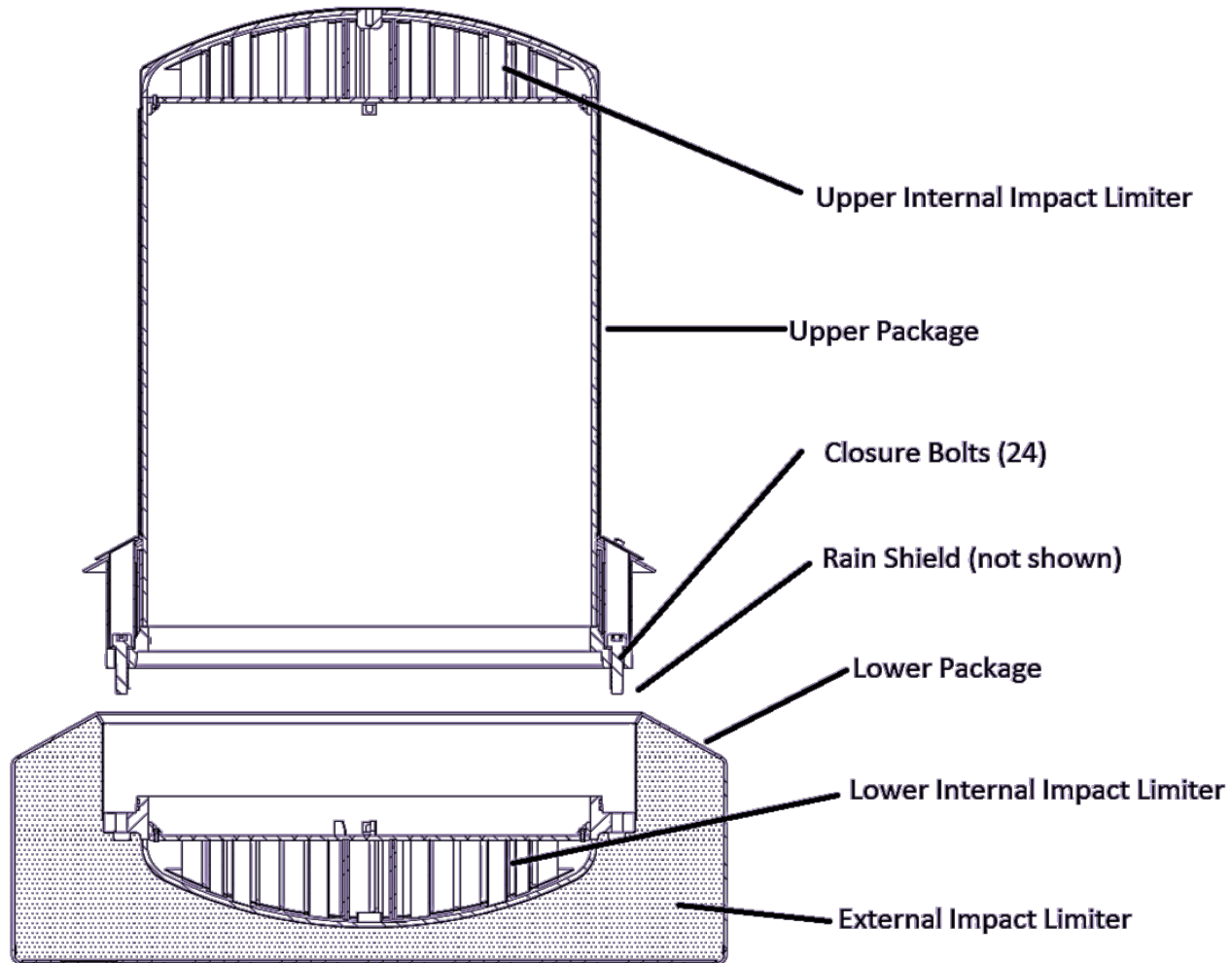


# 435-B Design

- Based on previously certified containers.
- Design criteria/parameters
  - Leak-tight – Normal Condition of Transport (NCT) and Hypothetical Accident Conditions (HAC)
  - Transportable by truck, rail, ship, air
  - External dimensions 211 cm H x 178 cm outside diameter (OD)
  - Internal Cavity 152 cm H x 110 cm inside diameter (ID)
  - Gross weight 4,490 kg (2,636 kg empty)
  - 480 TBq Co-60, 200 Watts



# 435-B Design

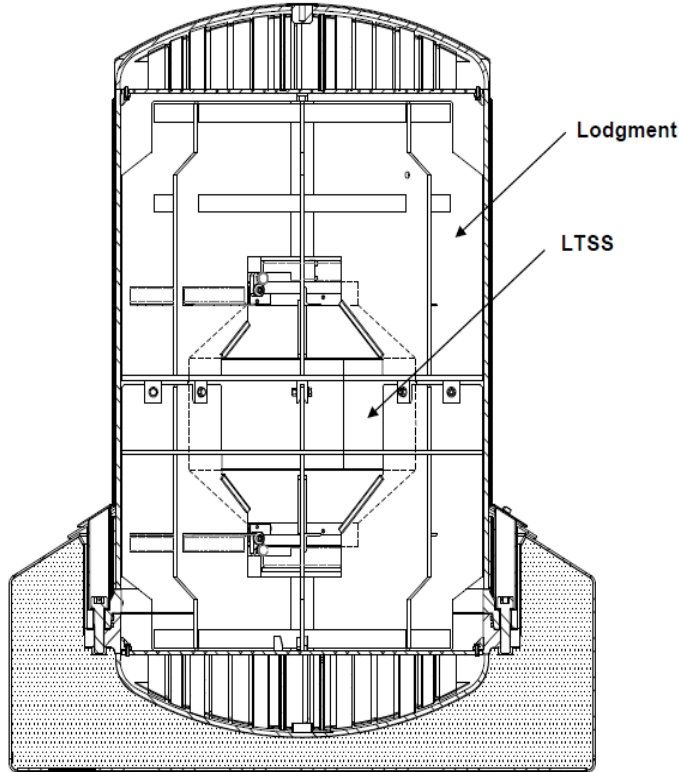


# 435-B Payloads

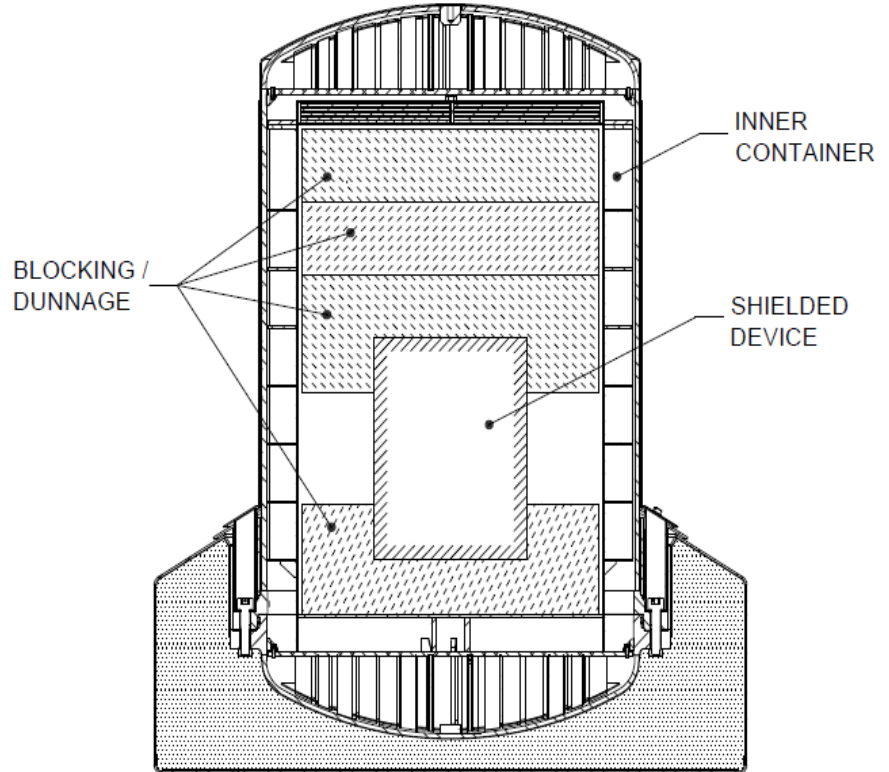
- For transportation of
  - The LTSS with a lodgment
  - The disposal canister with a lodgment
  - A large shielded device with a lodgment
  - A shielded device inside an inner container
- Maximum activities:

Nuclide	Maximum Activity (TBq)
Co-60	480
Cs-137	1000
Sr-90	37
Ra-226	0.7
Ra-226/Be	0.2
Ir-192	7
Se-75	3

# Cross-sectional View with Payloads

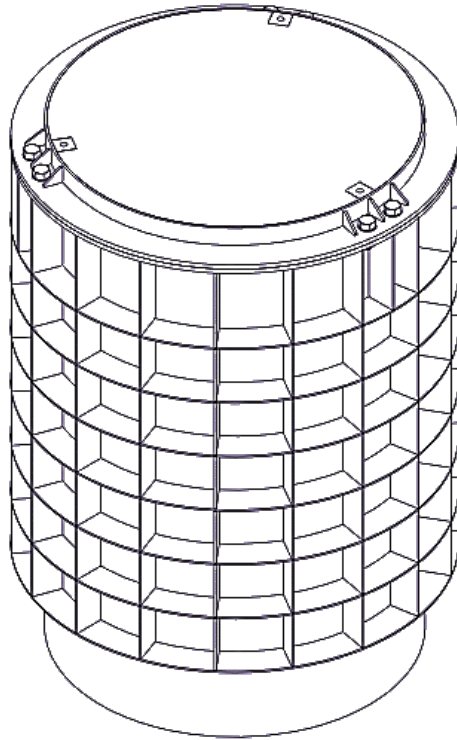


435-B with LTSS

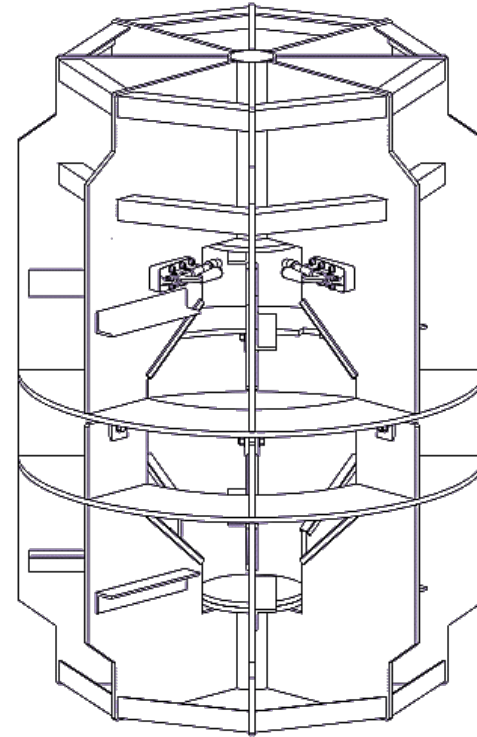


435-B with shielded device

# Bracing Components for Payload



Inner Container for Shielded  
Devices



LTSS Lodgement



# Authorized Content

- Because the 435-B packaging relies on the source retention and shielding properties of the payloads under both NCT and HAC, they must be evaluated prior to being added as authorized content.
- Current authorized content includes:
  - LTSS
  - Disposal canister
  - Gammator Series
  - Gammacell 1000
  - Gammacell 3000
  - Gammacell 40
  - IBL 437C
  - Hopewell Designs shielded devices



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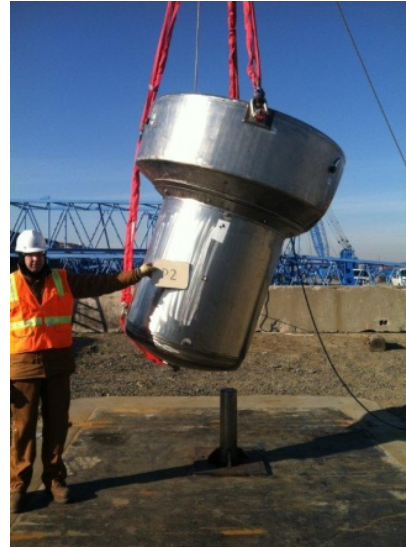


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# 435-B Testing



# Manufacturing the 435-B

- Fabrication started in November 2015
- Four units fabricated to date
  - Two are being operated by Idaho National Laboratory
  - One has been donated to the IAEA
  - One will be operated by Los Alamos National Laboratory

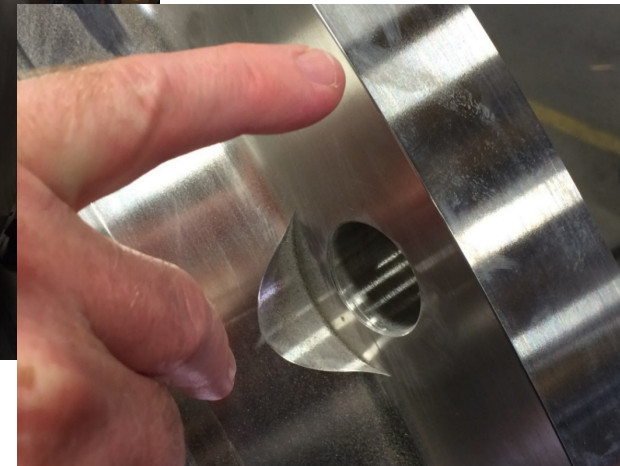
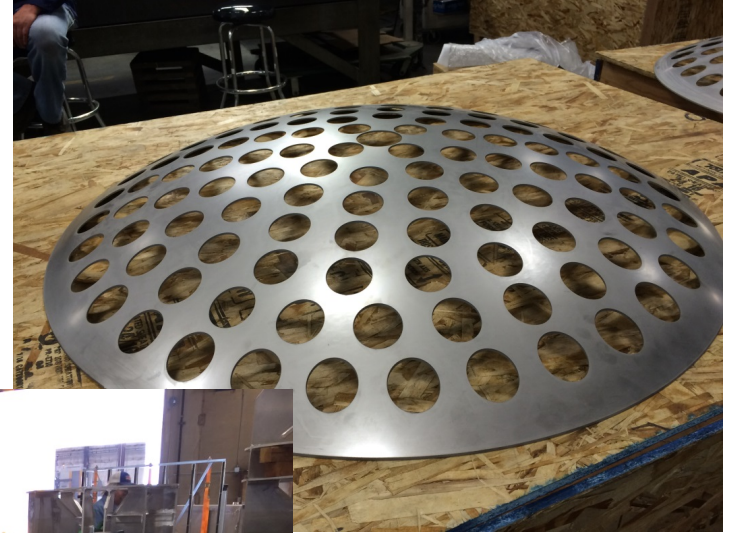




# Manufacturing the 435-B (continued)



# Manufacturing the 435-B (continued)



# 435-B Operations



# First Recovery: March 2018

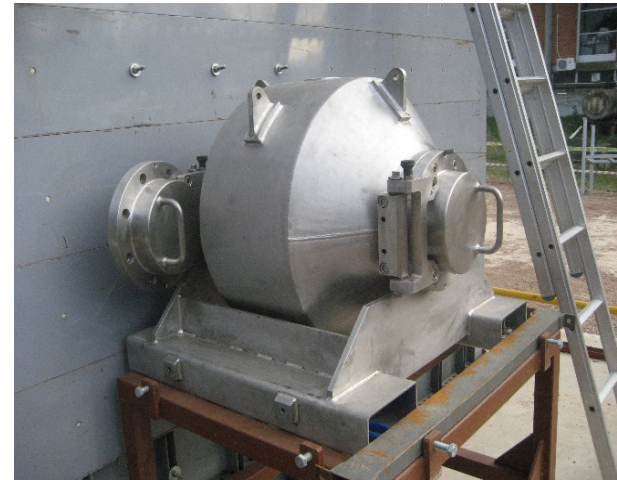


- In the early phases of container development, OSRP worked with the IAEA on design requirements
- International recoveries were anticipated using the IAEA mobile hot cell to disassemble and consolidate sources from high-activity devices
- The LTSS is designed to mate with the mobile hot cell to safely pull sources into the LTSS for transportation or storage

## IAEA Collaboration



IAEA's mobile hot cell



LTSS mated to mobile hot cell



# Questions?