

# Neutron-Dose Control of First Responders under Sampling and Categorization

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## Abstract

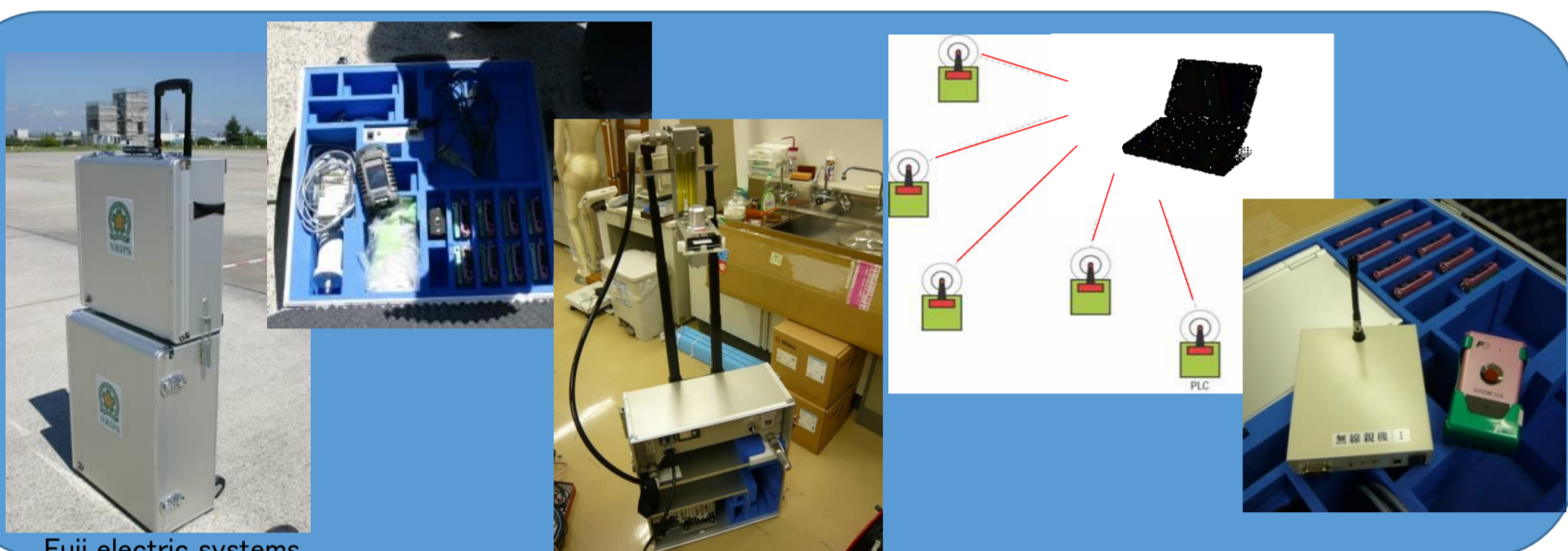
Nuclear materials and RDDs, which makes criticality field, emit neutrons whose energy range can vary from thermal to several MeV. In particular, the fast neutrons around over 1MeV have a strong damage for human body. Portable equipment and radiation protection for radiological emergency response team to achieve emergency tasks safely at the incident sites have been developed and evaluated in National Research Institute of Police Science (NRIPS, JAPAN). In this report, we introduce fast neutron shield with water and wireless network personal dosimeters under sampling and categorization. Described next in this report are evaluation tests of real-time neutron dosimeters using low-scatter room (neutron irradiation field) in Pacific Northwest National Laboratory. We evaluated them under fast neutron field and thermal neutron field.

## 1. Development of neutron shield and neutron dose monitoring system



### Neutron shield with water

The thickness of water shield in developed prototype equipment is 10cm, which decrease to 1/3 fast neutrons and 1/2 gamma-rays (Co-60). The neutron shield is mounted on an electric cart with DC motor, which maximum speed is 3km/h. A long tong is set to the center of shield, with which first responders can collect samples safely.



### 10 wireless network personal dosimeters

- Real time monitor system for personal dose with wireless network in the field.
- Maximum range : 100m at open space

**3 handheld devices** Radioisotope identification, neutron detection, high-dose rate and contamination check

**Air sampling** : 50L/min at 1m height

## 2. Past evaluation of personal dosimeters

### SILENE 2002 [1]

### Nuclear criticality field

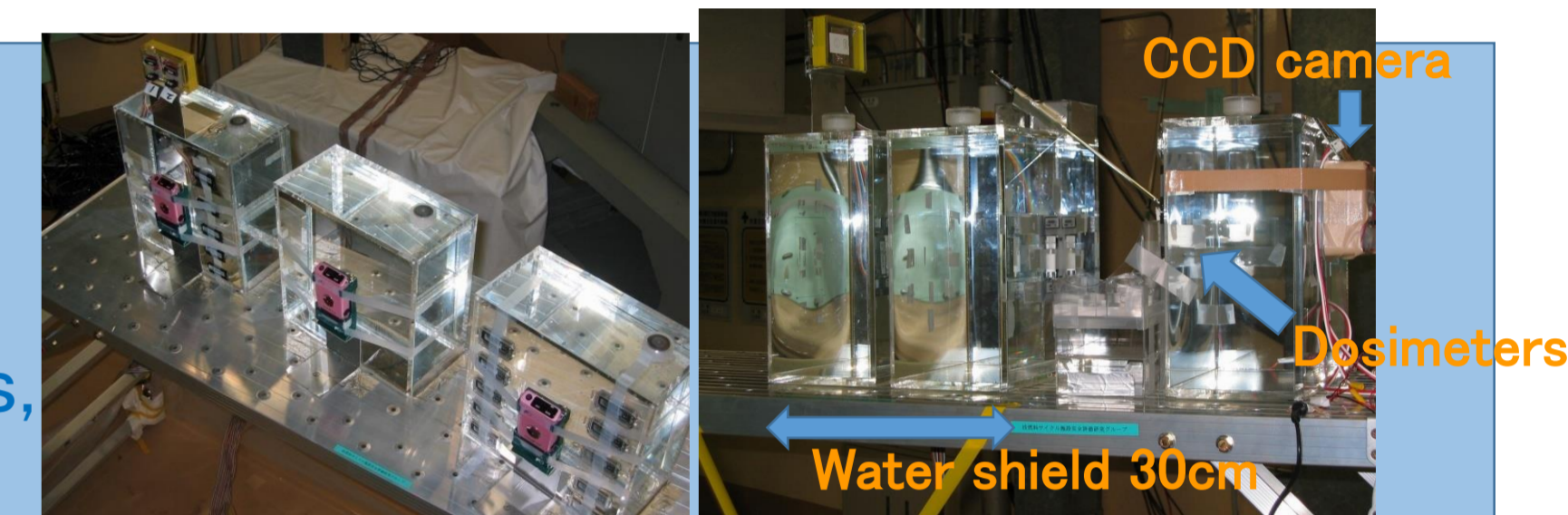
This reactor allows the simulation of various criticality accidents, pulse mode, free evolution and steady state. 71% of personal dosimeters are within  $\pm 25\%$  of reference dose value.



### TRACY(JAEA)-NRIPS 2010 [2]

### Nuclear criticality field

Tested are real-time personal neutron dosimeters of different types, NRG13 (Fuji Electric Systems, Japan), ADM-353(Aloka, Japan) and DMC2000GN (MGP, USA). These solid state dosimeters have digital displays of dose, and a warning function using light, sound and/or vibration. The dosimeters were attached on a 30\*30\*15cm<sup>3</sup> phantom, and located at the distances of 1.35m and 5.5m from the TRACY core. The dosimeters at 1.35m were irradiated directly (bare condition). On the other hand, a 30cm-thickness water shield was set in front of the dosimeters at 5.5m (water shield condition). The dose indicated by ADM-353 was 2 times by NRG13 in bare condition, whereas the dose indicated by ADM-353 was over 8 times by NRG13 in water shield condition(Figure.1). They are caused by the energy responses of dosimeters and variation of neutron spectra at the location.



Irradiation (1) Bare      Irradiation (2) Water shield (Displays on dosimeters were observed with a CCD camera)

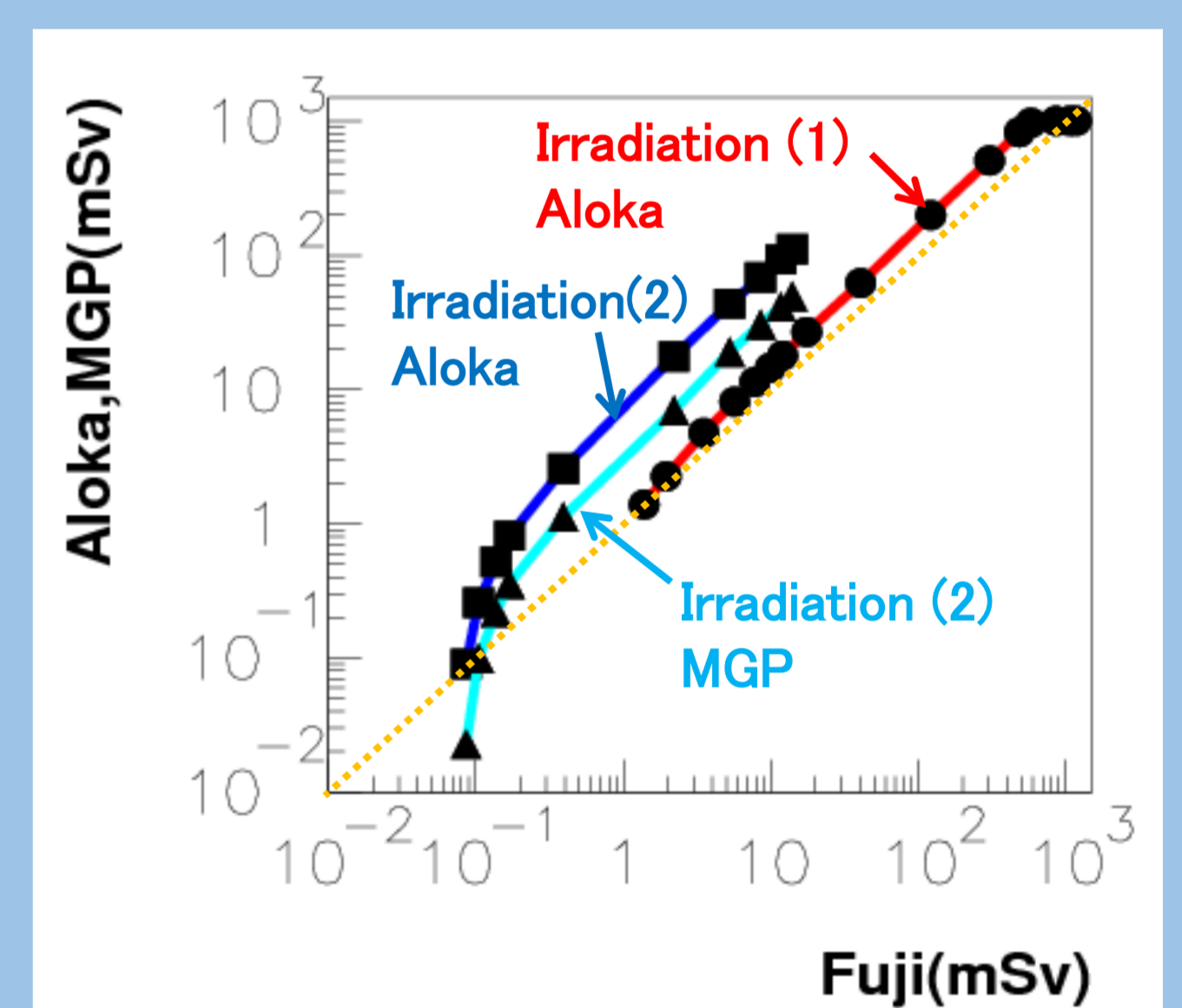


Fig.1 Comparisons between dose indicated by each dosimeters[2]. Aloka and MGP for Fuji Electric Systems

### Passive Dosimetry

### Active Dosimetry

## 3. Evaluation of neutron dosimeters

The dosimeters were attached on a 40 × 40 × 15cm<sup>3</sup> phantom and located at the distances of 30cm, 55cm and 100cm from the neutron sources (Am-Be, bare Cf-252, and moderated Cf-252), respectively. The dose rates were 0.866mSv/h, 3.41mSv/h and 3.68mSv/h, respectively.

We decided irradiation time in which the accumulated doses were set to about 0.5mSv.

We compared the response for each dosimeters under direct irradiation and irradiation with neutron shield (10cm thickness polyethylene corresponded to water). A 10cm-thickness shield was set 10cm in front of the dosimeters. The neutron spectrum with neutron shield was thermalized to lower energy, which intermediate neutrons increased.

	Aloka PDM313	Aloka ADM353	Thermo EPD N2	MGP(MIRION) DMC2000GN	Fuji NRF31	Fuji NRG13
Detector type	Neutron	Neutron/gamma	Neutron/gamma	Neutron/gamma	Neutron/gamma	Neutron/gamma
Resolution	10uSv	100uSv	1uSv	10uSv	1uSv	100uSv
Dose range	10uSv-100mSv	100uSv-1000mSv	1uSv-16Sv	10uSv-10Sv	1uSv-10Sv	300uSv-1000mSv
Size (mm)		110*52*18	86*63*19	87*48*31	60*78*27	
Weight (g)		90	108	80	<100	
# of dos.	#1,#2	#3,#4	#5-#7	#8-#10	#11-#13	#14

Table.1 Tested neutron dosimeters

Fig.2 Schematic view of evaluation geometry

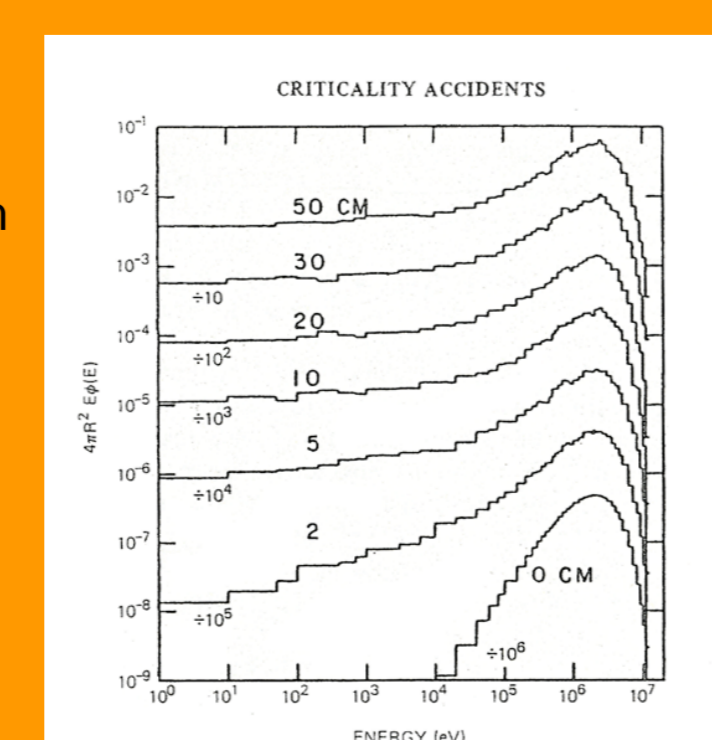
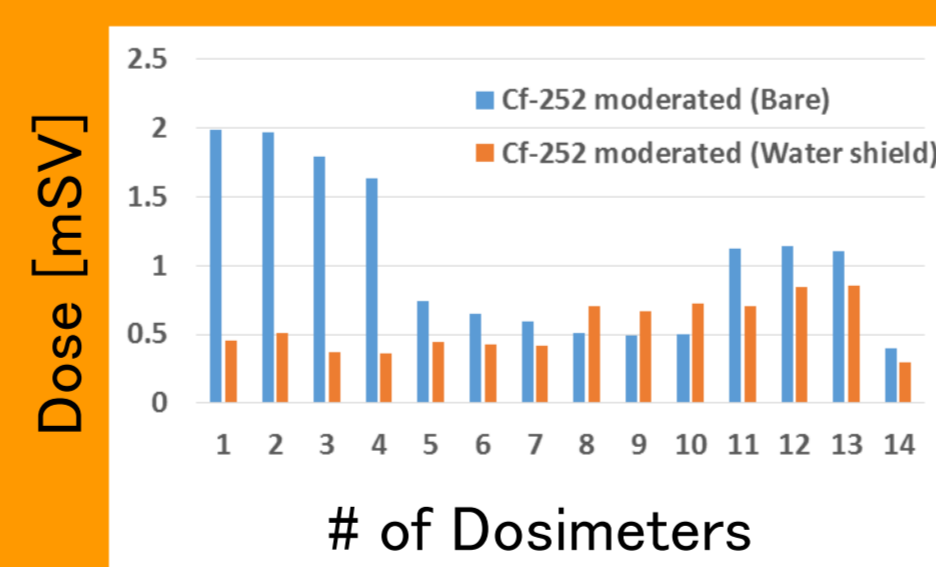


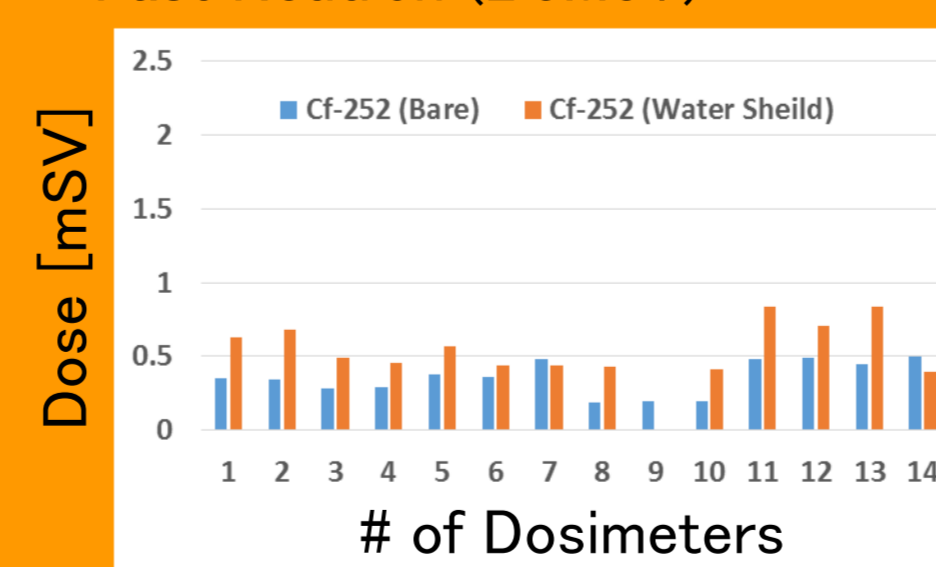
Fig.4 Neutron spectra penetrated in water. [IAEA Tec. Rep. Ser. No.211(1982)]

Thermal Neutron (around 0.025eV)



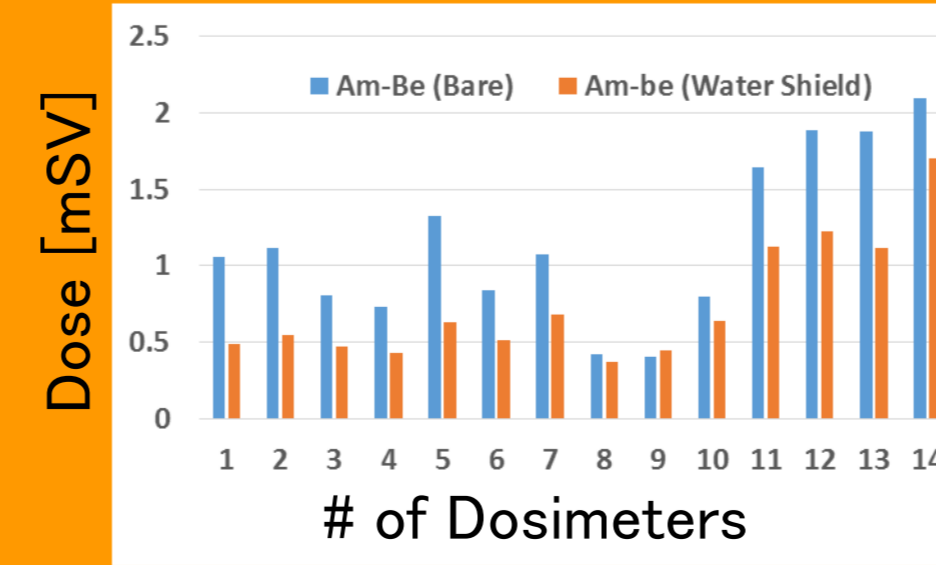
Field Neutron Dose (0.5mSv)

Fast Neutron (2.3MeV)



Field Neutron Dose (0.5mSv)

Fast Neutron (4.4MeV)



Field Neutron Dose (0.5mSv)

Fig.5 Response of different neutron Dosimeters.

## 4. Results and Summary

Under thermal neutron field (moderated Cf-252 source), responses of Aloka dosimeters were 2-4 times higher than those of other dosimeters. We can see same situation in the past results (at criticality field<sup>[2]</sup>, around a spent fuel cask<sup>[4]</sup>, EVIDOS Project<sup>[5]</sup>). However the significant difference of each dosimeter's responses was not confirmed under moderated Cf-252 source with neutron shield. Under Cf-252 irradiation which mean energy is 2.3MeV, doses of Aloka and Thermo dosimeters are lower than those of Fuji dosimeters, whereas the opposite tendency for Am-Be irradiation which mean energy is 4.4MeV. This is owe to the difference of calibration source (Cf-252 or Am-Be) for each dosimeters at factory setting. The difference is consistent with the past result<sup>[6]</sup>, which we lead to the correct dose using the correction factor for neutron energy.

### [Reference]

- [1]R. Medioni et al. "Criticality accident dosimetry systems: an international intercomparison at the silene reactor in 2002" Radiation Protection Dosimetry 110(1),429-436 ,2004
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- [4]S.Mayer et al. "Intercomparison of the response of different photon and neutron detectors around a spent fuel cask" Radiation Measurements 47, pp.634-639, 2012
- [5]M. Luszk-Bhadra et al. "Summary of Personal neutron Dosimeters Results Obtained within the EVIDOS Project" Radiation Protection Dosimetry Vol.125, No.1-4, Pp.293-299, 2007
- [6]K.Tsuchiya et al. "Radiation-induced failures and degradation of wireless real-time dosimeter under high-dose-rate irradiation" Proceedings of SPIE Defense, Security, and Sensing 2010 Vol.7665 76651G, 2010

Fig.3 Relative dose equivalent response for normally incident neutrons as a function of neutron energy[3].