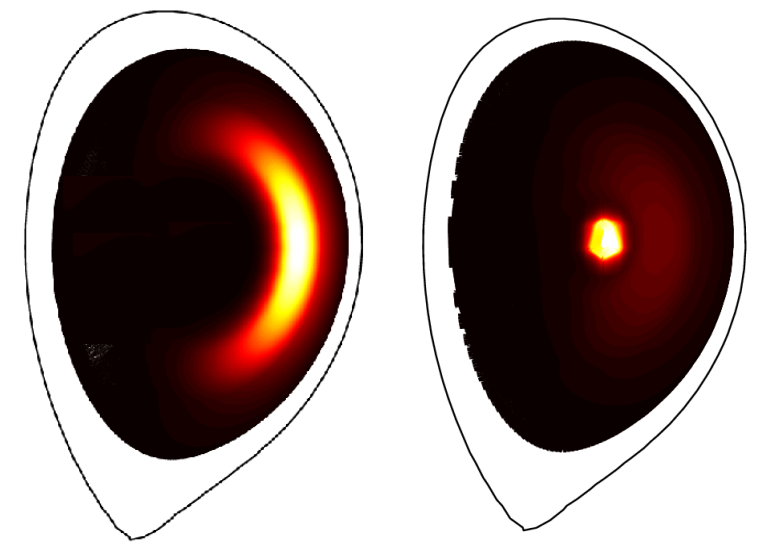


## Predictive multi-channel flux-driven modelling to optimise ICRH tungsten control in JET



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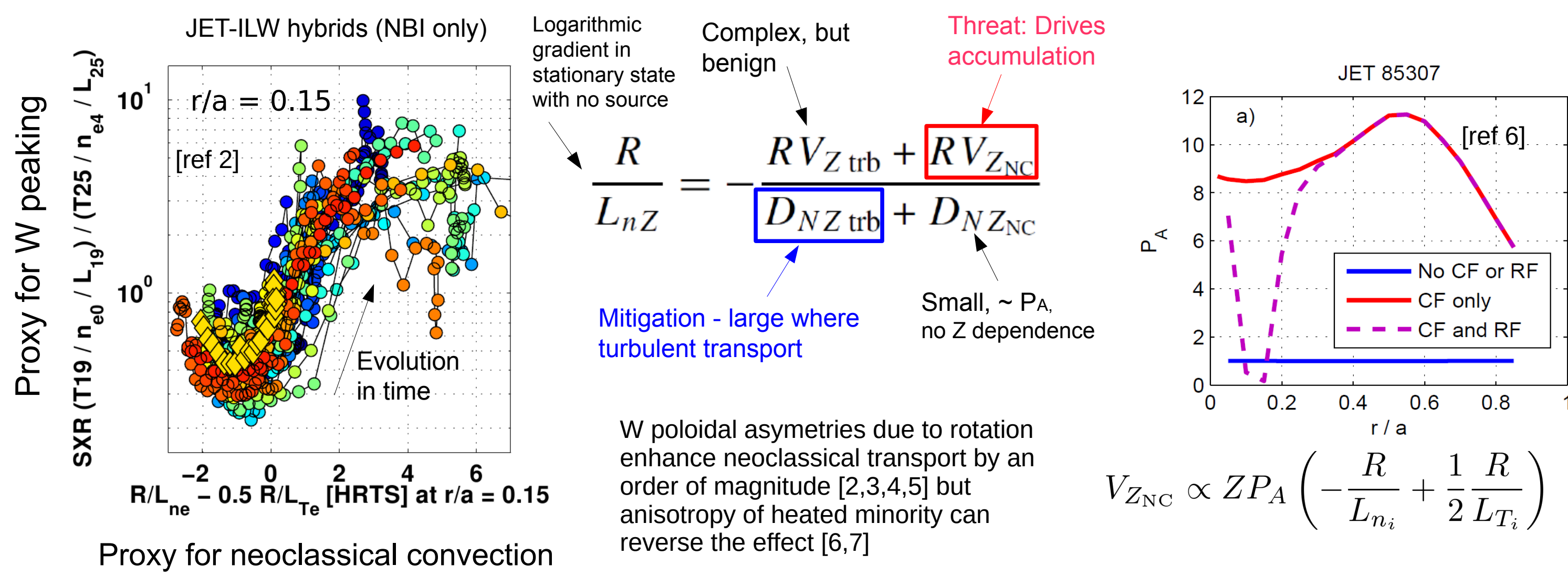
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\*See the author list of X. Litaudon et al 2017 Nucl. Fusion 57 102001

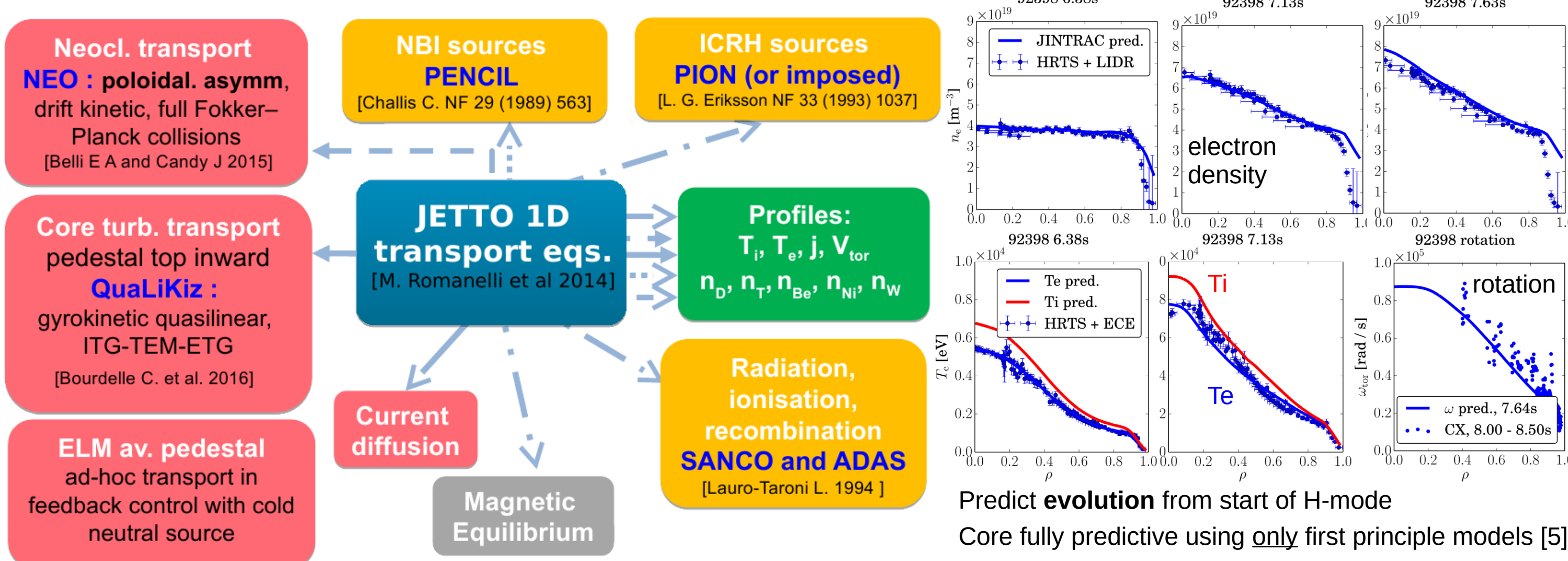
### Need to optimise JET-ILW scenarios against W accumulation

- JET-ILW DT scenarios aim at steady high performance (15MW fusion for 5s) [1]
  - Scenario development must address 3 connected challenges
    - Maintain tolerable divertor heat loads
    - Control central W accumulation
    - Avoid performance limiting MHD
- Apply state-of-the-art modelling capabilities to guide scenario development

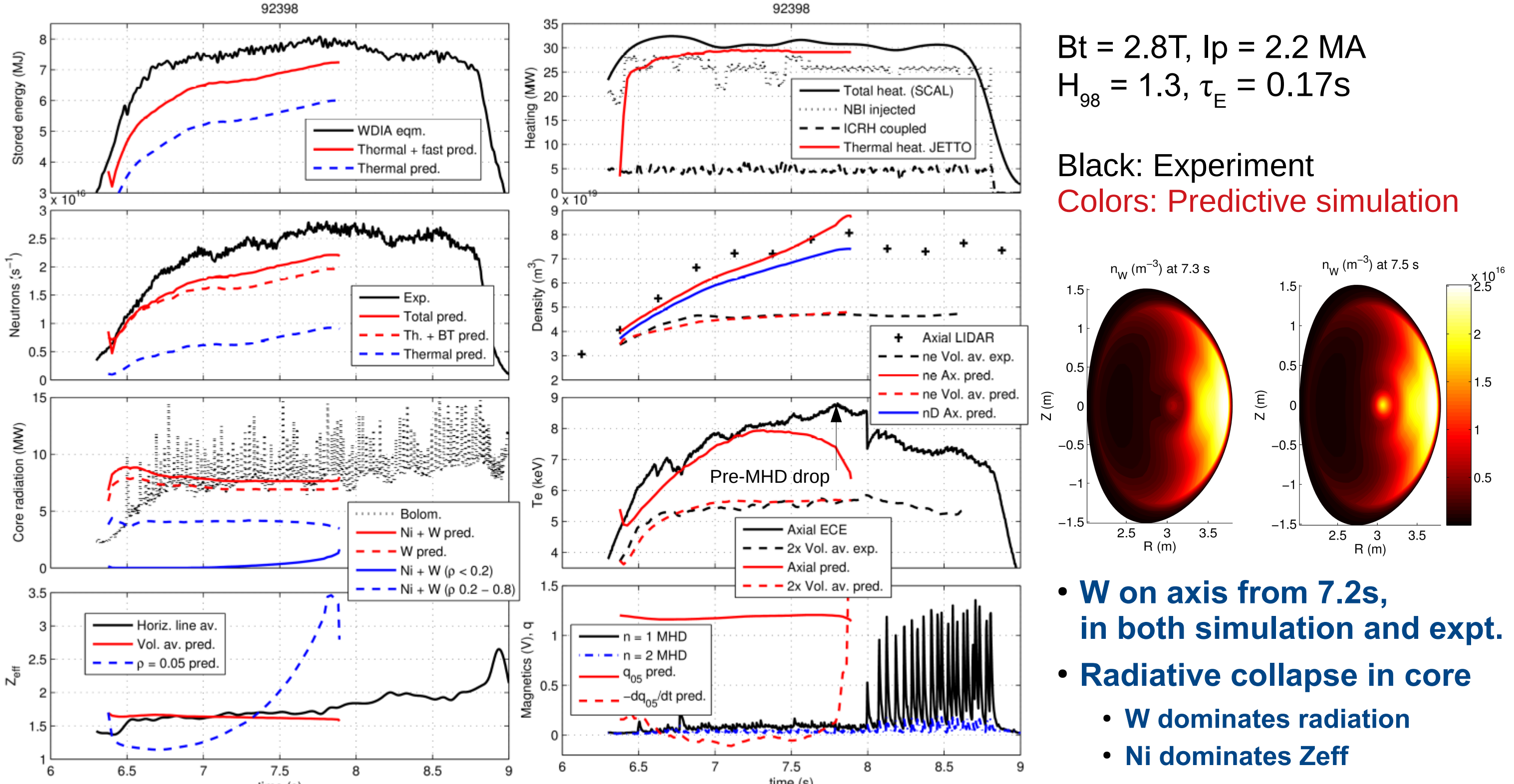
### W accumulation driven by neoclassical convection enhanced by rotation



### Integrate first-principle models to predict 9 channels self-consistently



### Validation of modelling against highest performance hybrid discharge

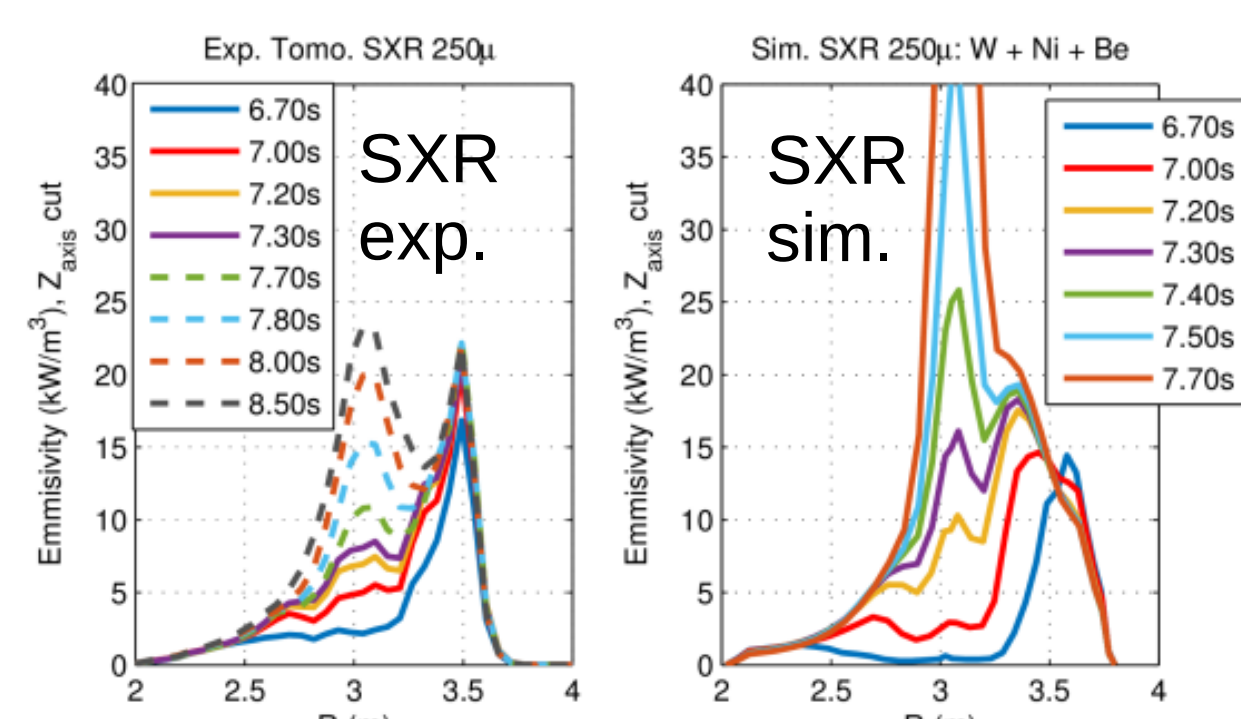


### Accumulation process more controlled in expt.

- Simulations extremely sensitive in accum. phase
- May suggest a missing transport process
- No ad-hoc transport used

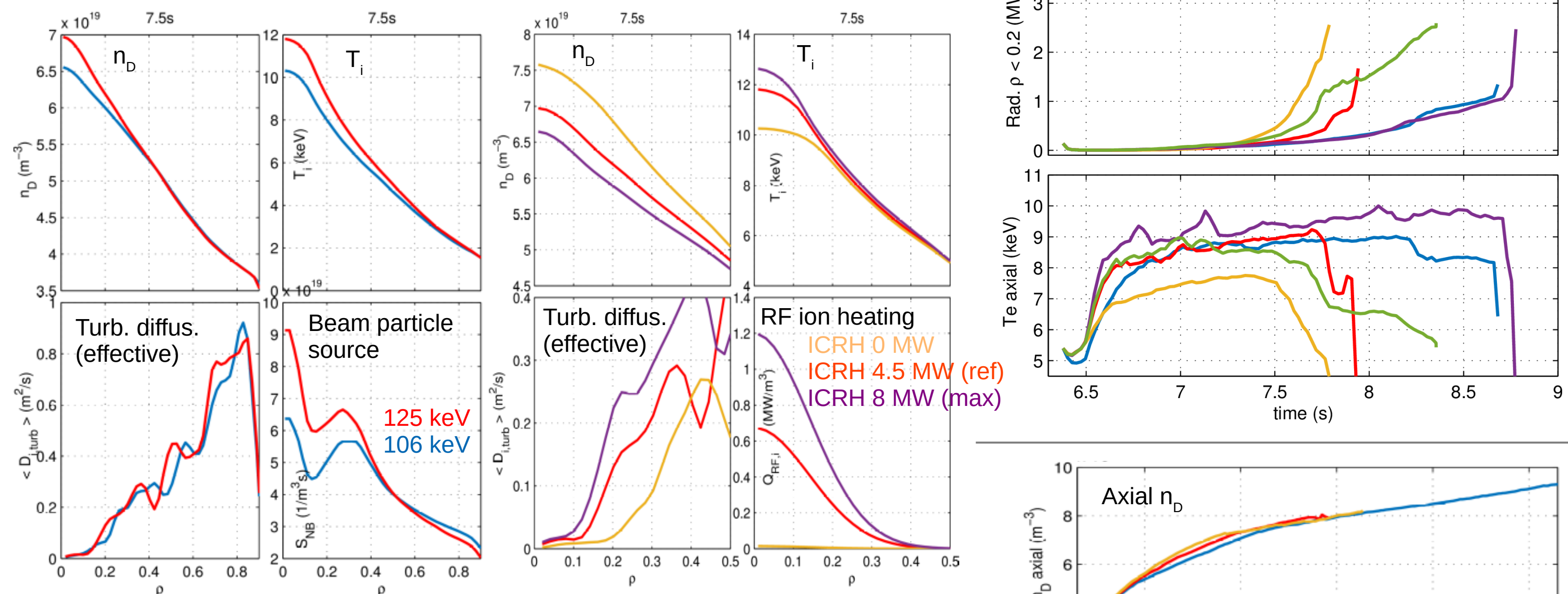
### 1,1 MHD arrives after accumulation begins

- Triggered by W? → loss of central bootstrap current
- Limits performance but mitigates accumulation



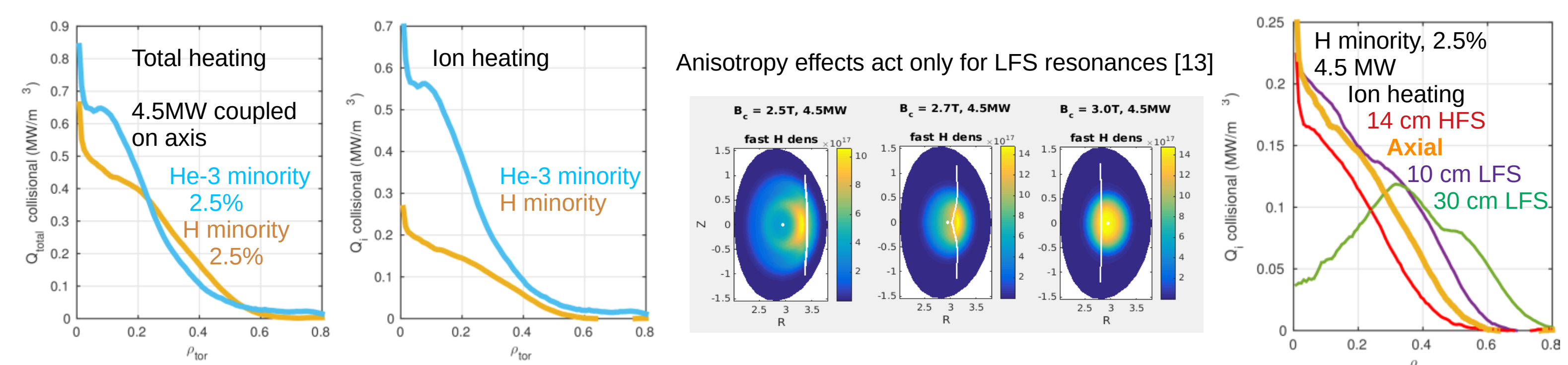
### Optimisation of heating against W

- Increased NBI power will accelerate W accumulation [5]
  - Beam energies will be increased to reach maximum power
  - More central power, particle (density peaking), and torque deposition
- ICRH helps most in neoclassical dominated core, both increasing ∇T<sub>i</sub> and decreasing ∇n<sub>D</sub>
  - Increased turbulent diffusion reduces central density peaking localised axial ICRH most effective
  - Predictions consistent with JET observations [8,9]
  - 4MW increase in ICRH compensates 6MW increase in NBI



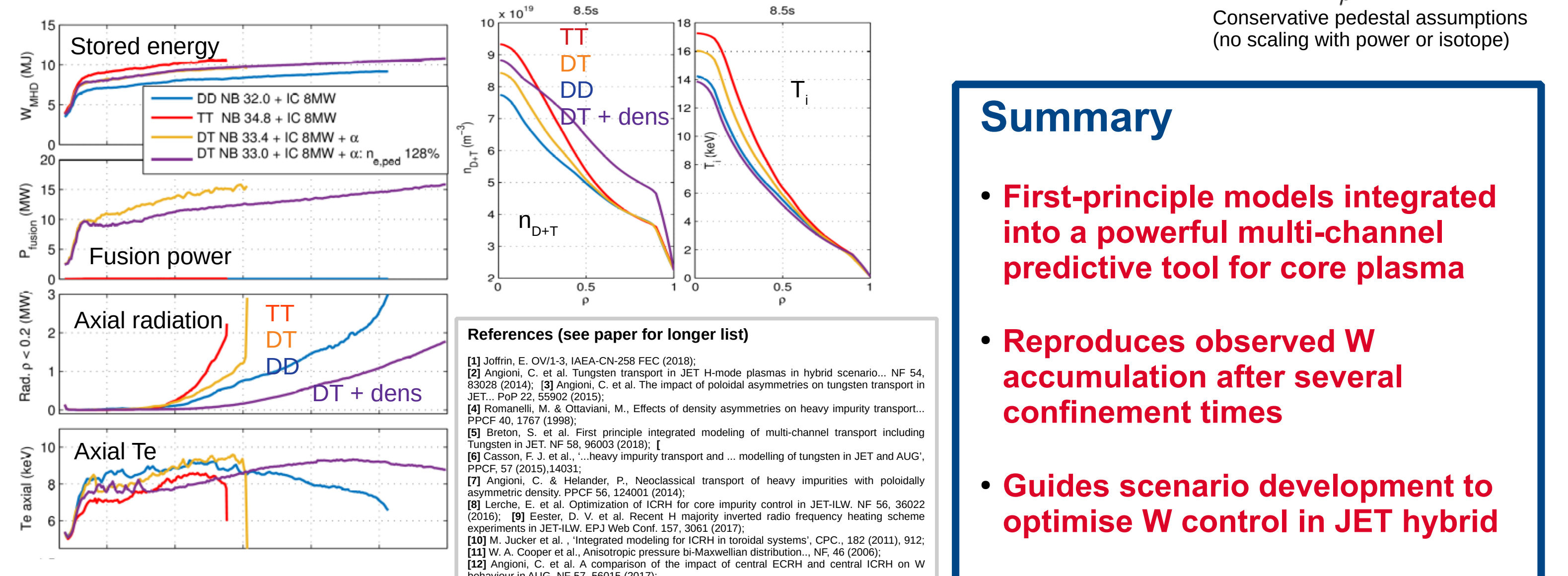
### Ion heating schemes predicted as most effective on W

- Ion heating both increases ∇T<sub>i</sub> and decreases ∇n<sub>D</sub>
  - Specific to JET hybrid scenario: T<sub>i</sub> > T<sub>e</sub>, dominant neoclassical convection (large Mach no ~ 0.7)
  - Where T<sub>i</sub> ~ T<sub>e</sub> coupled, or turb dominates, electron heating best [12] (e.g. ITER)
- Supporting high-fidelity ICRH model (SCENIC [10,11]) shows
- He-3 minority scheme achieves larger power density and larger ion heating
  - Resonance within 10cm of axis is optimal; fast ion effects on W transport negligible with FOW



### Predictions for DT

- Tritium plasmas have better confinement but earlier W accumulation
  - Inclusion of ETG scales pins Te; i-e collisional energy exchange reduces with mass
  - Increased T<sub>i</sub> / T<sub>e</sub> and ITG stabilisation; specific to plasmas with T<sub>i</sub> > T<sub>e</sub> [14]
  - Improved confinement in DT also gives larger density peaking
- Mitigate with increased density (less central NBI particle deposition)
  - Some cost in performance - requires optimisation / integration
  - Access via increased triangularity / plasma current / pedestal isotope scaling



### Summary

- First-principle models integrated into a powerful multi-channel predictive tool for core plasma
- Reproduces observed W accumulation after several confinement times
- Guides scenario development to optimise W control in JET hybrid