

International Conference on Fast Reactors and Related Fuel Cycles: Next Generation Nuclear Systems for Sustainable Development (FR17)

Wednesday, June 28, 2017

Poster Session 2 (5:50 PM - 7:00 PM)

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[109] Analyses of unprotected transients in GFR (ALLEGRO) and SFR reactors supporting the group constant generation methodology	Mr BATKI, Bálint	
[103] Potential Capabilities in Transmutation of Minor Actinides of the BOR-60 Reactor and MBIR Reactor under Construction	Mr ZHEMKOV, Igor	
[213] Experiment and Analysis of Flow distribution of MOX Assembly	Mr ZHU, Huanjun	
[0] Nuclear Reactor Modelling and Simulation Toolkit (NuReMoST) –Numerical Reactor Model Configuration System with Interface to Simulation Codes	Mr BATRA, Chirayu	
[346] Development and Applications of Nuclear Design and Safety Assessment Program SuperMC for Fast Reactor	Prof. HU, liqin	
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[270] Computational Analysis Code Development for Emergency Heat Removal of Pool-style Fast Reactors	Ms XUE, Xiuli	
[271] Concept of multifunctional fast neutron research reactor (MBIR) core with metal (U-Pu-Zr)-fuel	Mr BLANDINSKIY, Victor	
[273] Development of innovating Na leak detector on pipes	Ms GIRARD, Marianne	
[277] Inspection specifications leading to extended ASTRID Design rules	Mr BAQUE, Francois	
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[446] DESIGN VALIDATION OF PFBR FUEL SUBASSEMBLY TRANSPORTATION CASK WITH MOCKUP TRIAL RUN	Mr PILLAI, Puthiyavinayagam	
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[105] Calculation and Experimental Data Analysis of Neutron Spatial/Energy Distribution in the BOR-60 Blanket	Mr NABOYSHCHIKOV, Yury	
[39] Design and Fabrication of Closed Loop Systems (CLS) for the Fast Flux Test Facility (FFTF)	Mr WOOTAN, David	
[36] ANALYSIS OF VARIOUS APPROXIMATIONS IN NEUTRONIC CALCULATIONS OF TRANSIENT IN FAST REACTORS	Mrs CHERNOVA, Irina	
[437] Features of the physics of the MBIR reactor core	Mr ELISEEV, Vladimir	
[18] Modeling of Phenix End-of-Life control rod withdrawal tests with the Serpent-DYN3D code system	Dr FRIDMAN, Emil	

[91] Development and Validation of EBRDYN code by Benchmark Analysis of EBR-II SHRT-17 Test	Mr UPPALA, Partha Sarathy	
[551] Application of CFD simulation to validate the BREST-OD-300 primary circuit design	Mr TUTUKIN, Alexey	
[552] LES-SIMULATION OF HEAT TRANSFER IN A TURBULENT PIPE	Mr SERGEENKO, Konstantin	
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[50] Low-void-effect sodium-cooled core: Uncertainty of local sodium void reactivity as a result of nuclear data uncertainties	Dr PELLONI, Sandro	
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