

# International Conference on Fast Reactors and Related Fuel Cycles: Next Generation Nuclear Systems for Sustainable Development (FR17)



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## Computational Analysis Code Development for core and primary system thermal hydraulic design of SFR

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This study developed a thermal hydraulic design code based on powerful capabilities of computer hardware and advanced numerical simulation technology for the optimization design of reactor core assemblies, big and small grid plates and the primary circuit fluid network, for the use of core assemblies thermal hydraulic design, special design of flow distribution in the big and small grid plates, as well as the design of complex throttle network in the primary circuit of fast reactors, providing a necessary tool for engineering design while supporting safety analysis of the reactor core and the primary circuit.

### Country/Int. Organization

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