

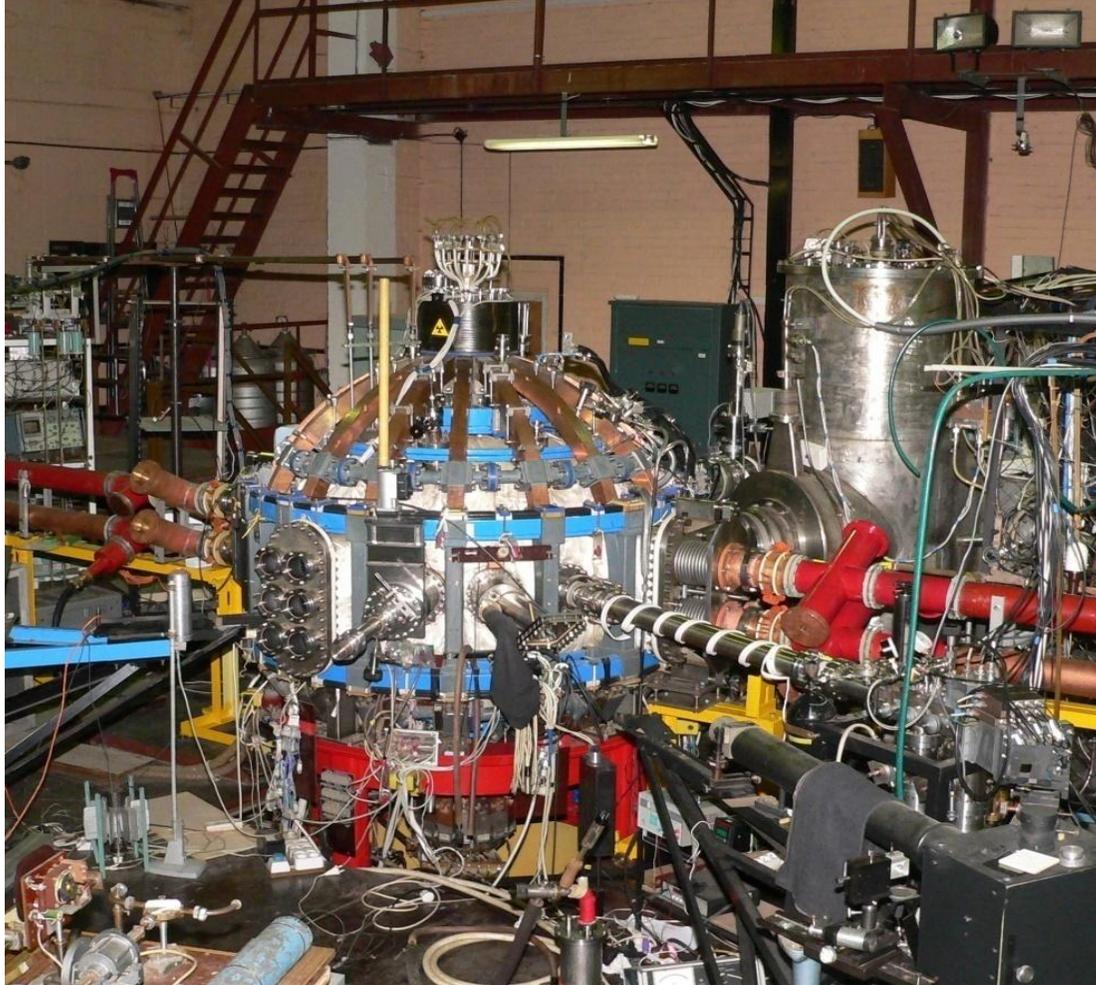
# Globus-M Results Toward Compact Spherical Tokamak with enhanced Parameters Globus-M2

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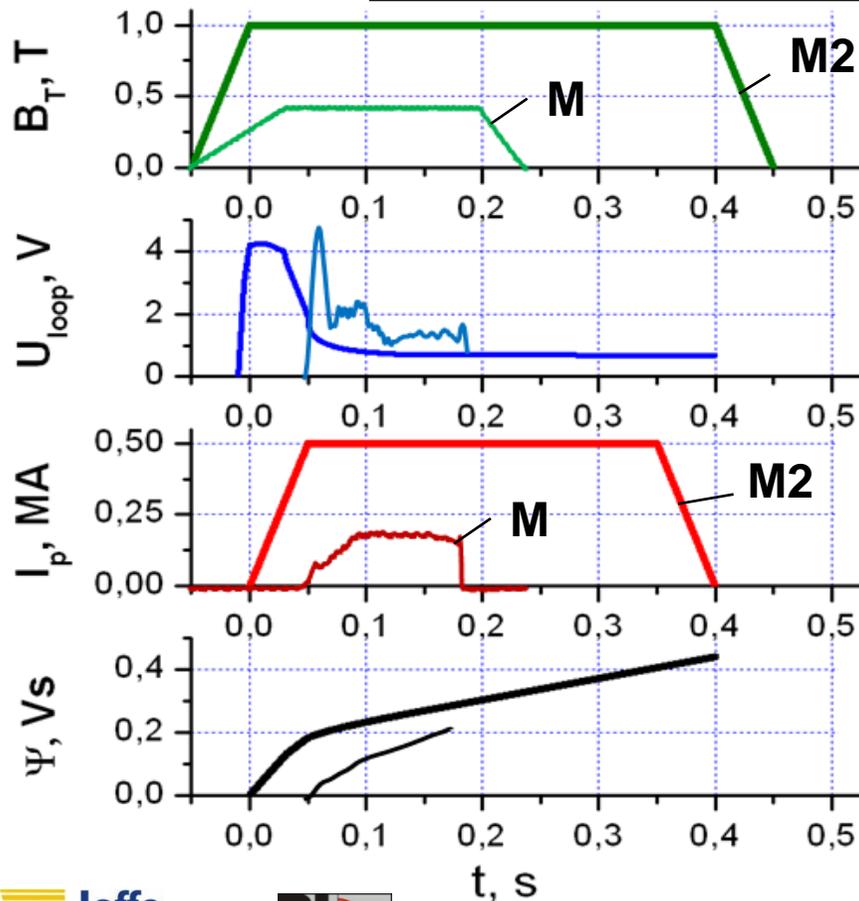
# Globus-M spherical tokamak demonstrated practically all of the project objectives



R	0.36 m
a	0.24 m
R/a	1.5
$B_{tor}$	0.4 T
$I_{pl}$	< 0.36 MA
$\kappa$	< 2.0
$\delta$	< 0.5
$\beta_{tor}$	<15 %
$\beta_N$	<6.0 %mT/MA
$T_{pulse}$	<130 ms
NBI [E/P]	30 keV / 1 MW
ICRH [F/P]	10 MHz / 0.3 MW
$T_e$ [max]	~1 keV
$T_i$ [max]	~0.8 keV
$q_{95}$ [min]	>2
$B_{tor}/R$	<1.5 T/m
$\langle n \rangle$ [max]	<1.2 · 10 <sup>20</sup> m <sup>-3</sup>

Maximal  $T$  and  $n$  values were obtained in different regimes

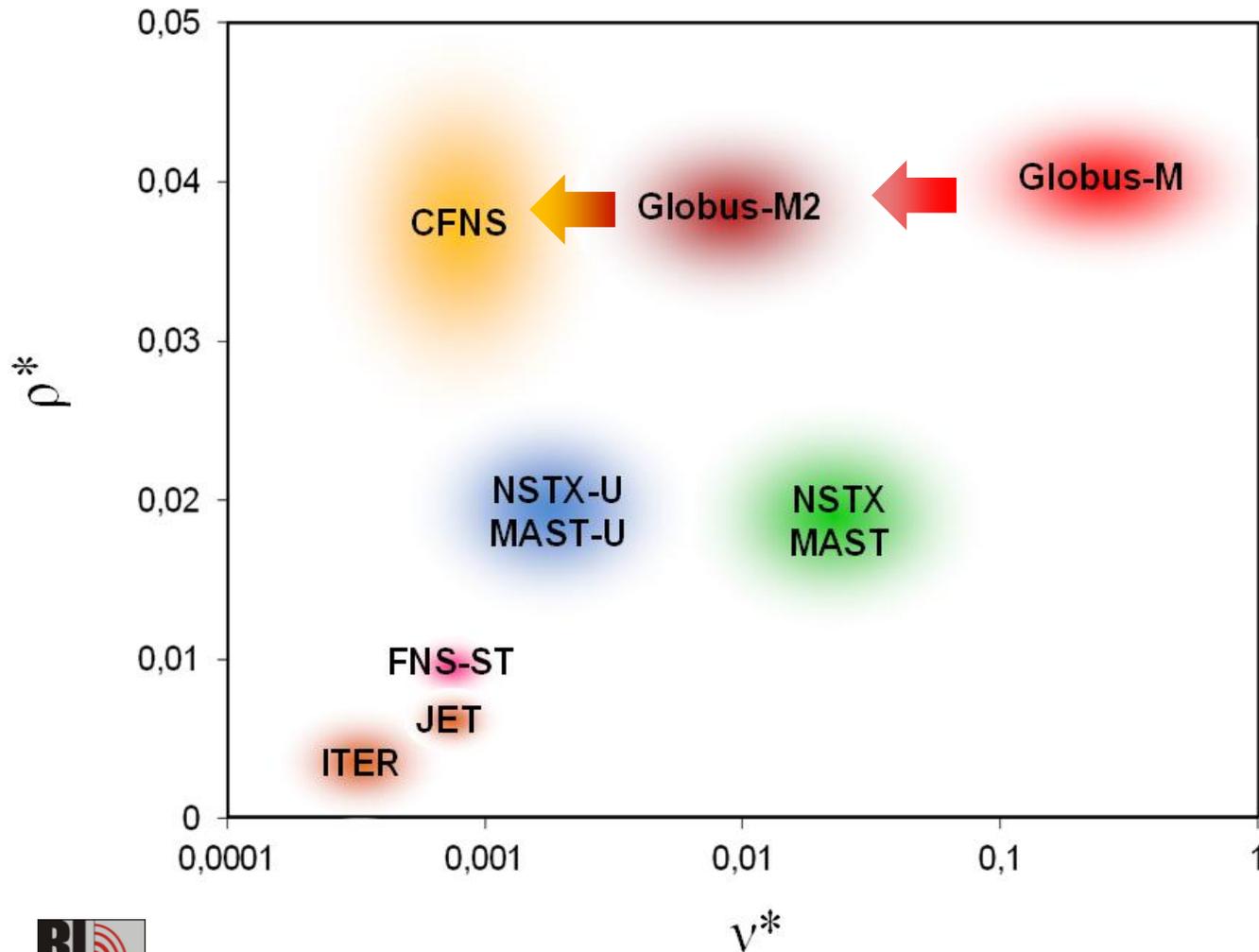
**We propose the simplest way to improve plasma parameters: both magnetic field and plasma current 2.5 increase retaining other machine parts unchanged.  
The machine name is “Globus-M2” ST**



**“B-max”** regime providing:  
 $B_T(R=0.36) = 1 \text{ T}$  for  $\Delta t = 0.4 \text{ sec}$ ,  
 $I_p = 0.5 \text{ MA}$  for 0.3 sec

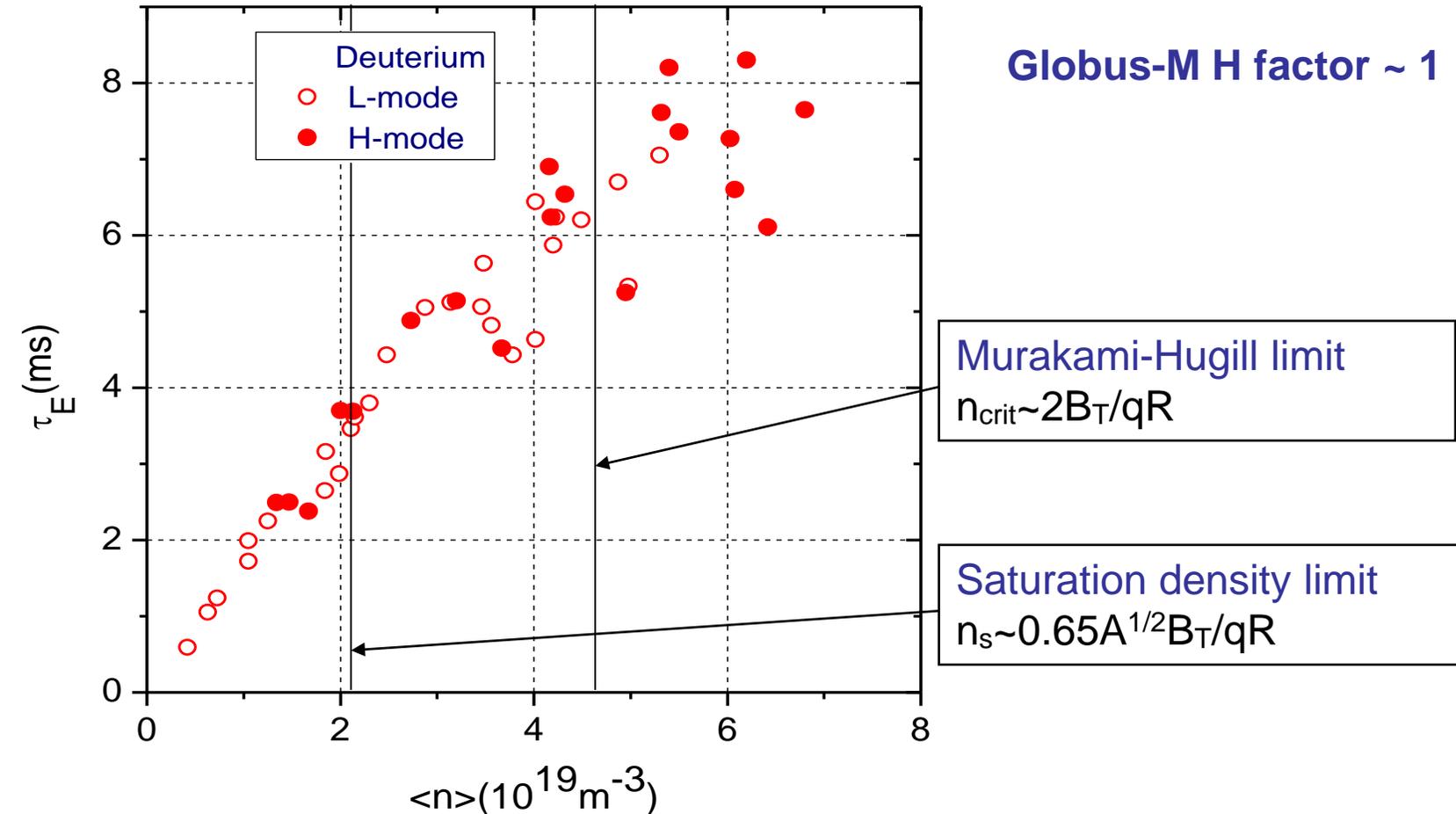
**“t-max”** regime for noninductive  
current drive experiments  
providing:  
 $B_T(R=0.36) = 0.7 \text{ T}$  for  $\Delta t = 0.8 \text{ sec}$

# Objectives of Globus-M2 permit regime modeling for Compact Fusion Neutron Source (CNFS)



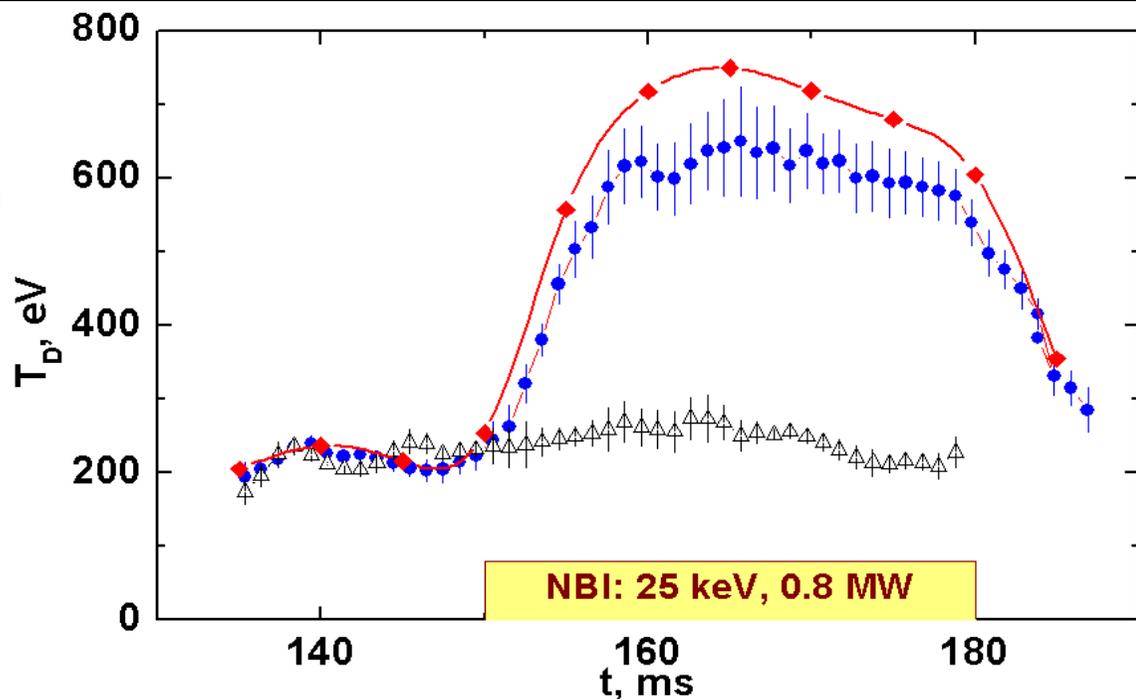
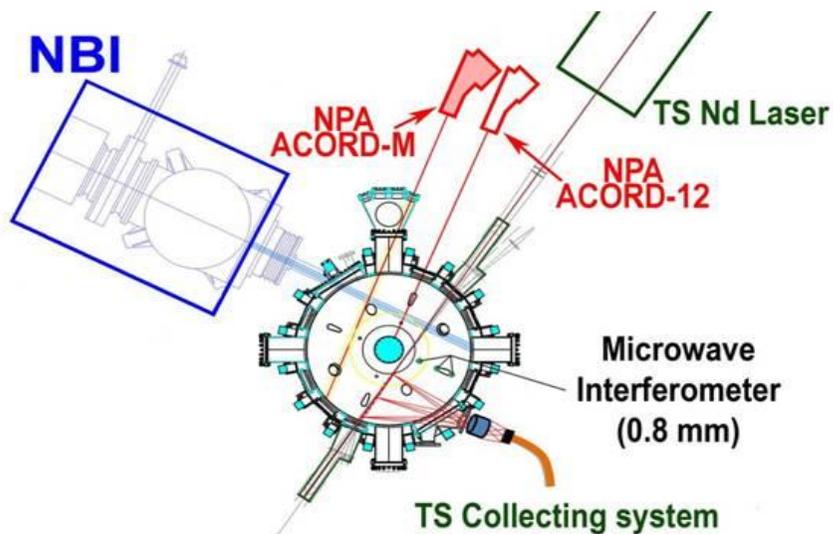
# **Globus-M confinement, heating, fast particle physics and edge plasma issues**

# Ohmic L and H-mode confinement



Linear lifetime dependence is characteristic for small size machines with domination of electron heat transport, e.g. Alcator-A, ISX-B, etc

# NB injection is the major instrument for plasma auxiliary heating in Globus-M

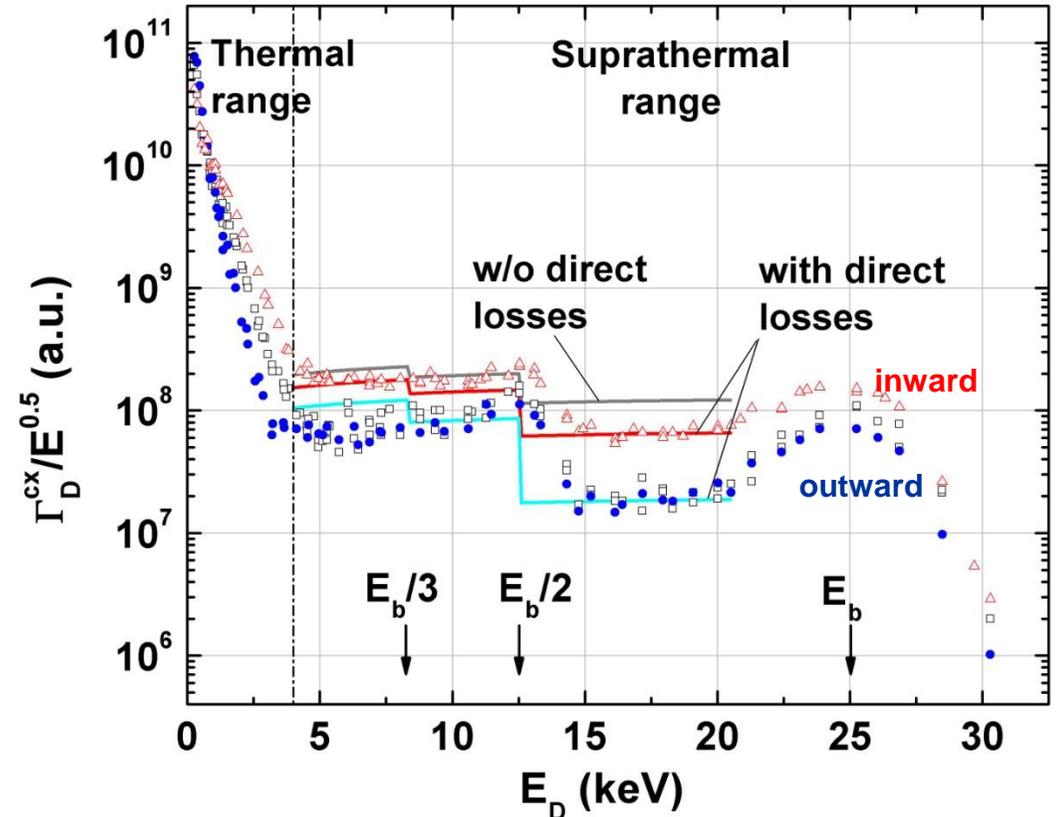
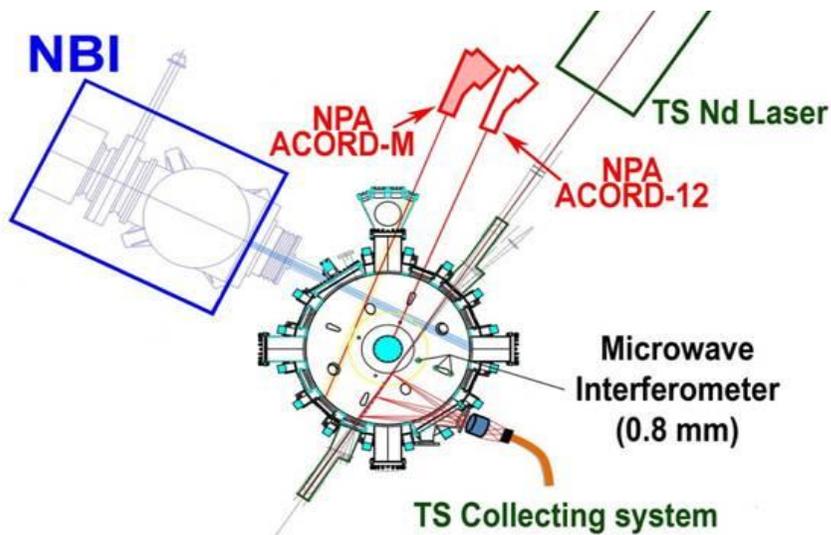


**In spite of difficulties connected with a compact size of the machine good ion heating efficiency during NB injection observed at low density.**

$$(2-4) \cdot 10^{19} \text{ m}^{-3}$$

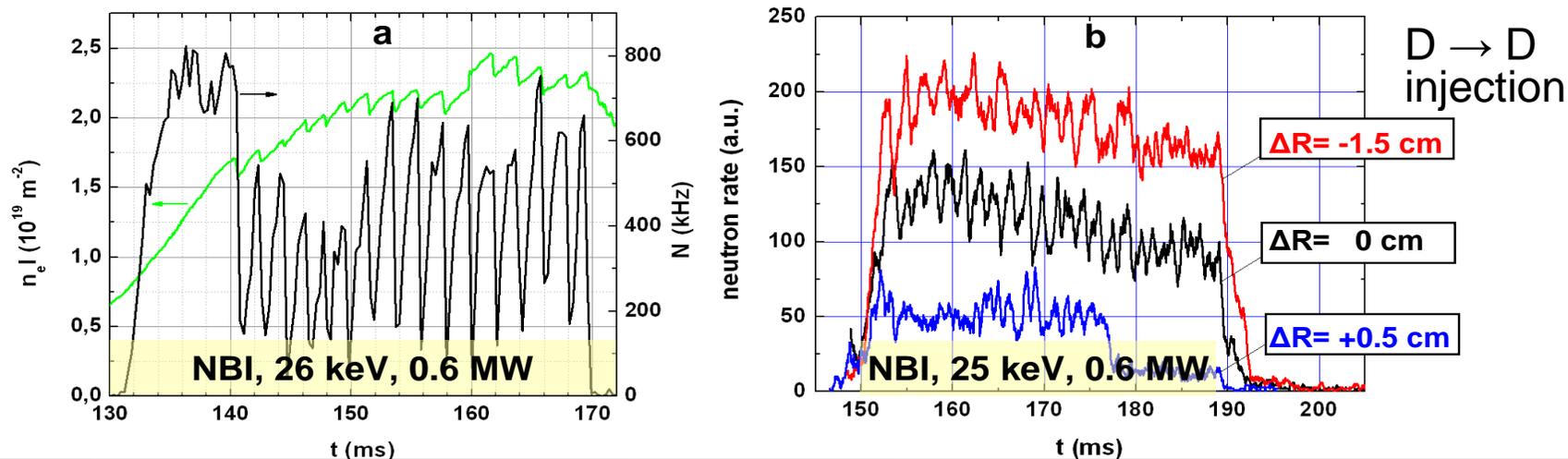
$$\chi_i \sim \chi_{i,neo}$$

# Fast particle first orbit losses at various major radii

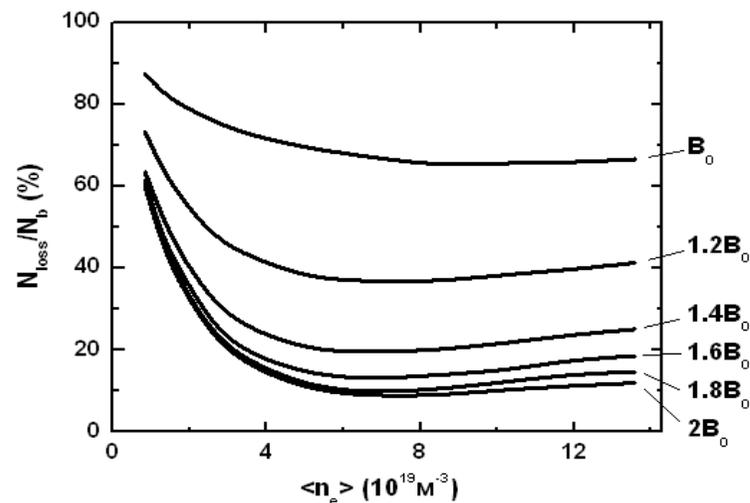


- “Horizontal” lines approximate Fokker-Planck equation solution with slowing down losses and allow for direct losses estimate
- Direct fast particle losses is the main mechanism deteriorating NB plasma heating in Globus-M
- “Bump on tail” – demonstrates additional source of losses

# Sawteeth are responsible for additional fast particle direct losses, producing “bump on tail” distribution

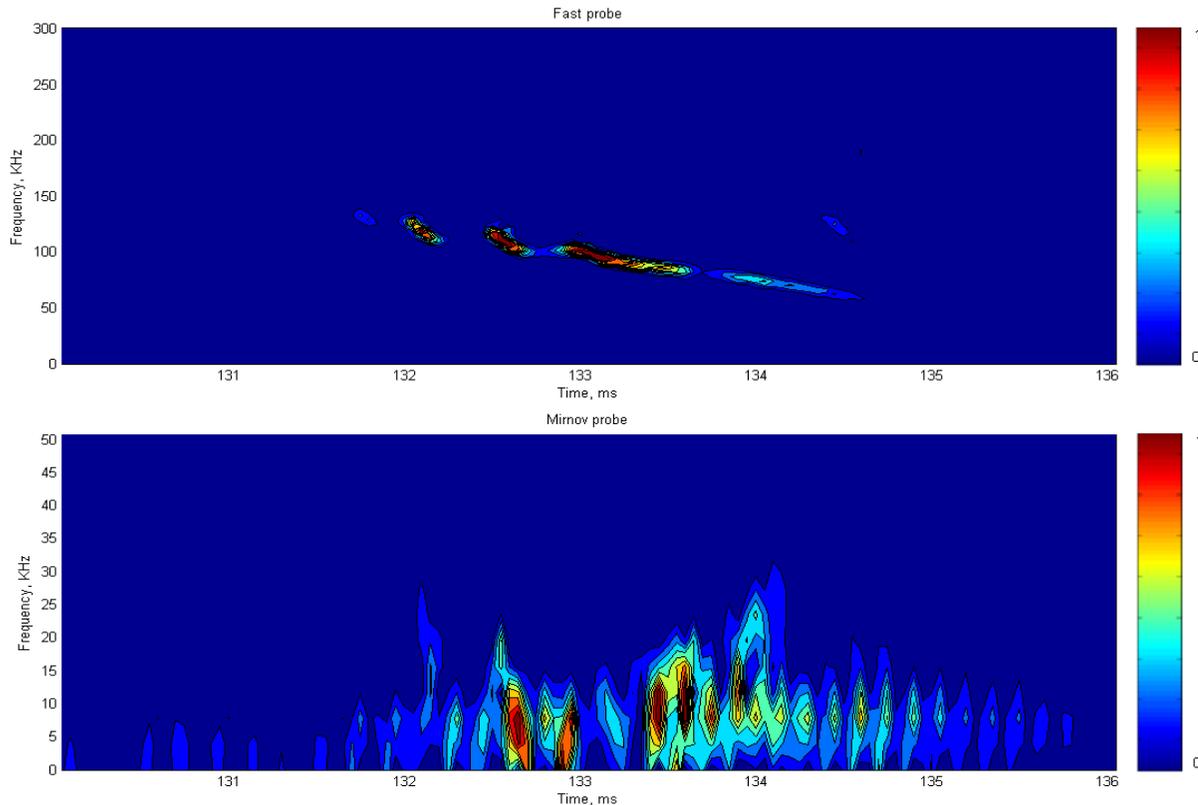


**Drastic decrease of fast particle losses can be achieved by total magnetic field increase**



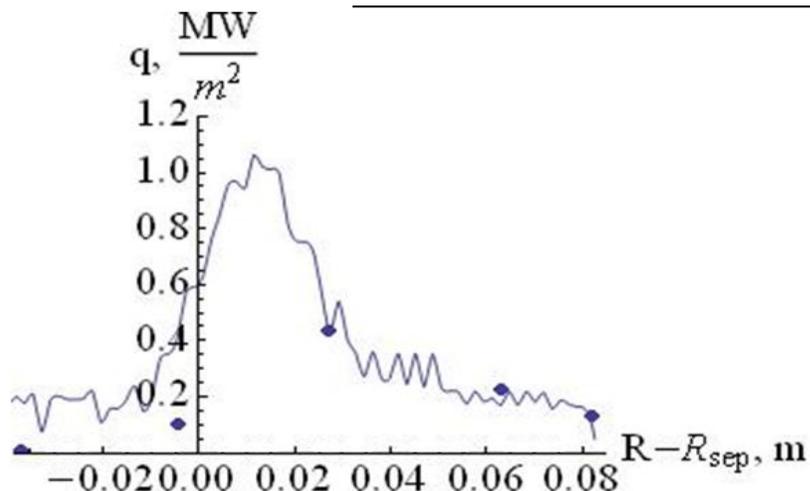
# TAE single n=1 mode excitation

H-NBI into D-plasma, E= 27 keV



- **Magnetic field 2.5 fold increase make it possible 6 fold beam energy increase without TAE spectrum enrichment:**  
 $V_b/V_A \approx 1$  on Globus-M  
 $V_b/V_A \sim \sqrt{E_b/B}$  at  $n_e = \text{const}$
- **Unlike TAE EPM is recorded as multimode excitation**
- **No additional Fast particle losses – evidence is neutron flux measurement**
- **Expectations for Globus-M2 are favourable**

# Prospects for edge plasma parameters is important issue for the upgraded machine

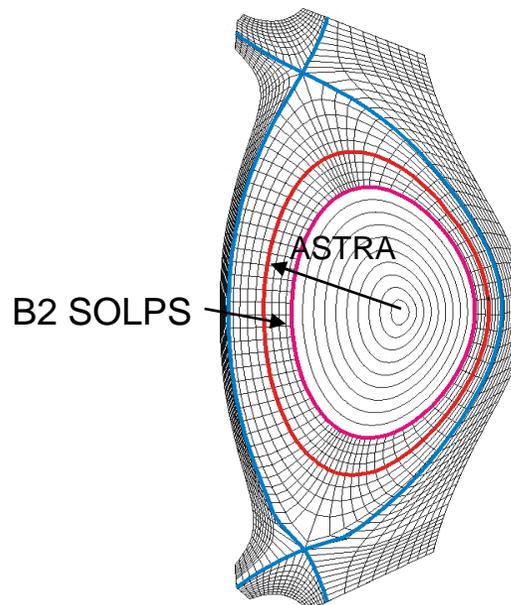
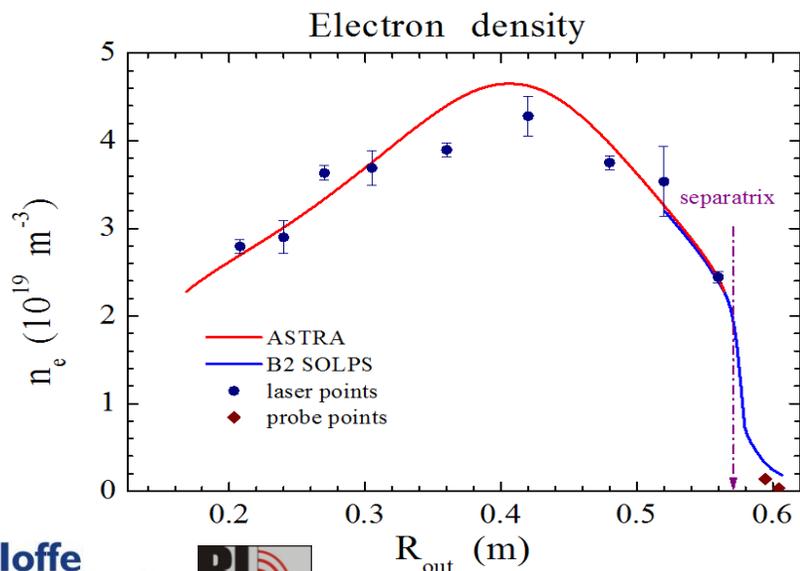


## IR videocamera and Langmuir probes data:

SOL heat flux e-folding length,  $\lambda_{qMP} \approx 3-5$  mm  
 SOL divertor plates width,  $\lambda_{qD} \approx 15-20$  mm

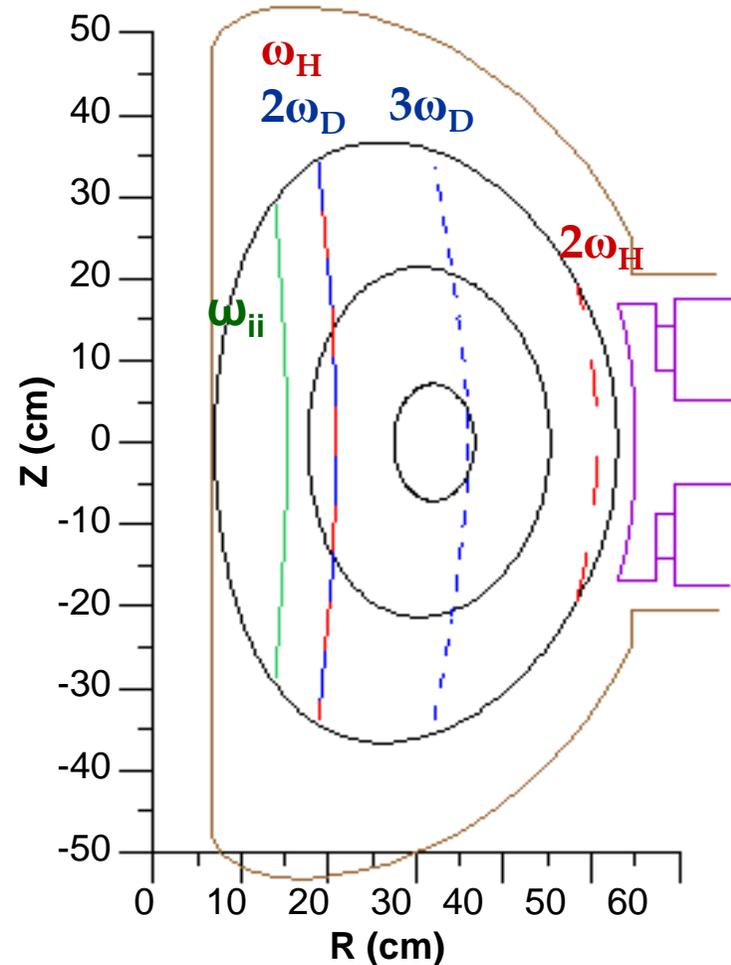
$P_{max}/S \approx 1.1$  MW/m<sup>2</sup> (OH LSN discharge)

$$\lambda_{qMP}(1T) = 2a\rho/R \sim 2a\sqrt{T_i}/RB_{pol} \sim \lambda_{qMP}(0.4T)$$



# ICRH prospects for 1 T in Globus-M2

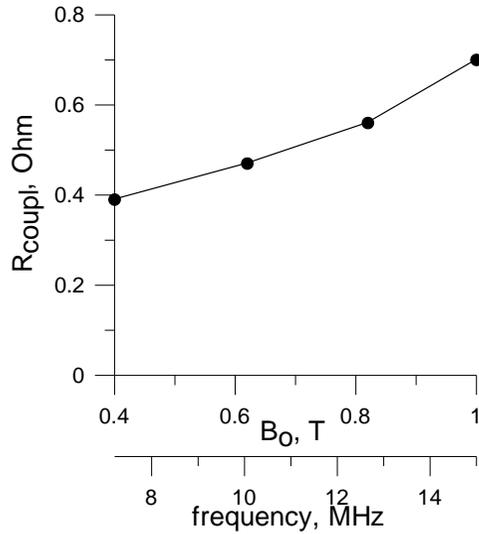
# If $B_T$ is increased from 0.4 to 1 T what will be benefits for ICRH heating by FMS waves at fundamental harmonic range?



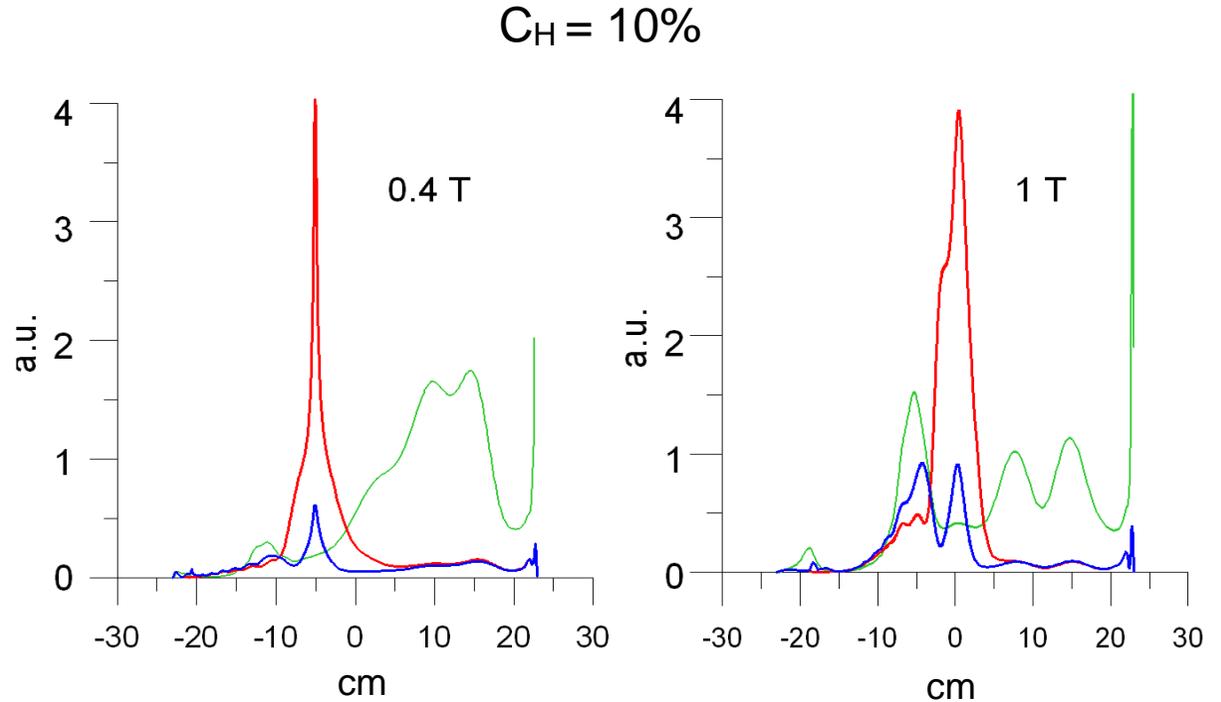
$B_T$	0.4T	1 T
$1f_{ci}$ for $H^+$	6.1 MHz	15.2 MHz

- Single pass resonance absorption of FMS waves increasing due to  $\lambda$  decrease
- Ion-ion hybrid resonance absorption increasing
- High magnetic field improves fast ion confinement and heating efficiency

# ICR heating efficiency increases with magnetic field raise from 0.4 T to 1 T



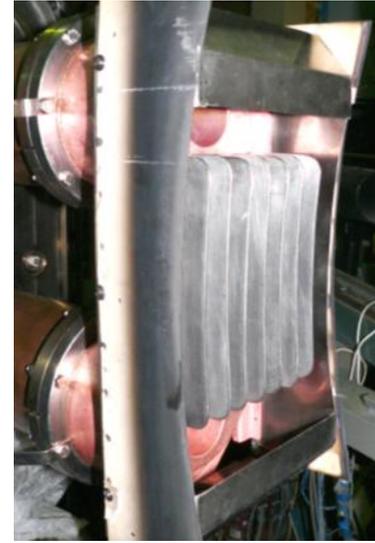
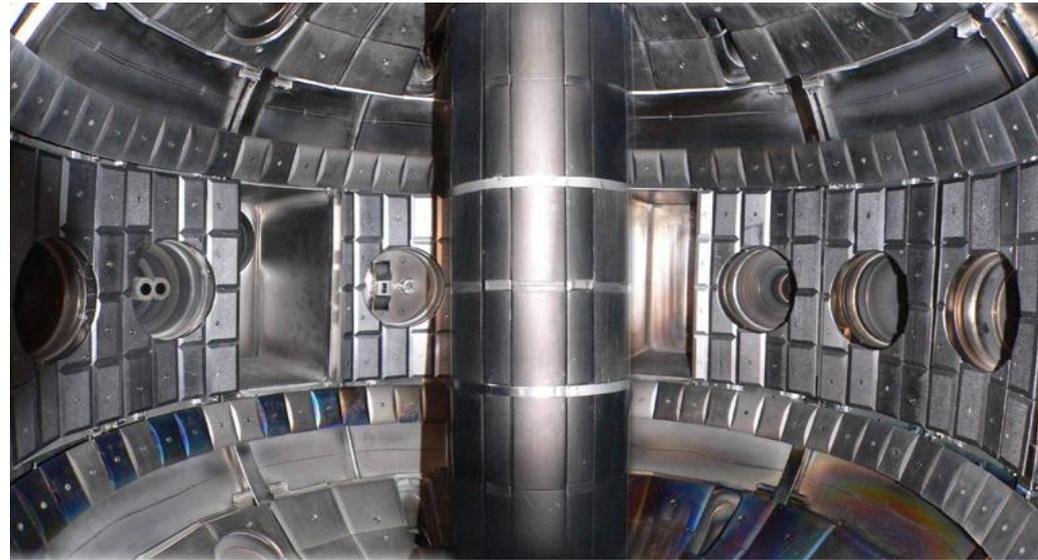
*Antenna-plasma coupling resistance raise increases RF power absorption efficiency with magnetic field*



Particle	Fraction of power at 0.4 T	Fraction of power at 1.0 T
$e^-$	0,641	0.46
$p^+$	0,260	0,38
$d^+$	0,099	0,16

# LH CD experiment in Globus-M and expectations for Globus-M2

# Novel experimental arrangement was used in experiment with plasma breakdown, current ramp-up and sustainment by LH waves



**NEW antennae with  
"toroidal" slow down:**

$$N_{tor} \approx (6 - 7)$$

$$N_{pol} \approx 1$$

**Vessel walls were totally  
protected by graphite**

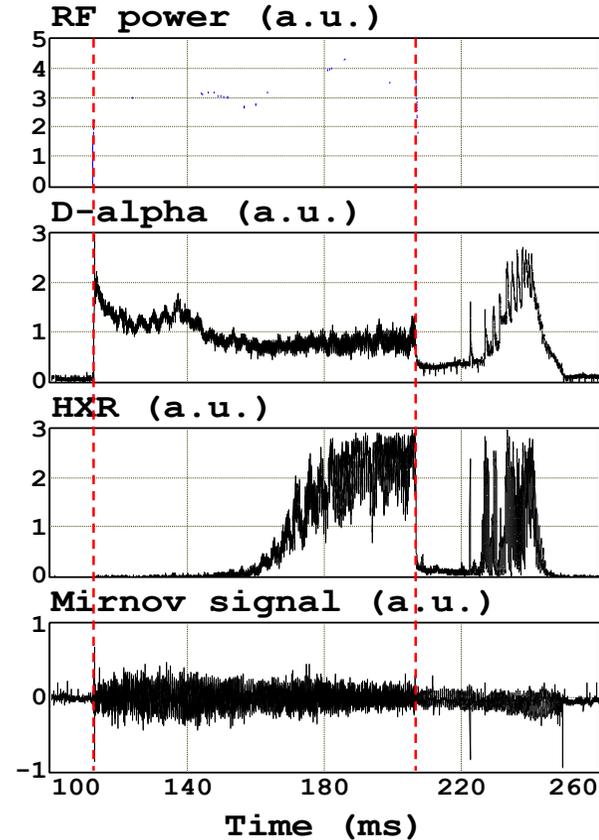
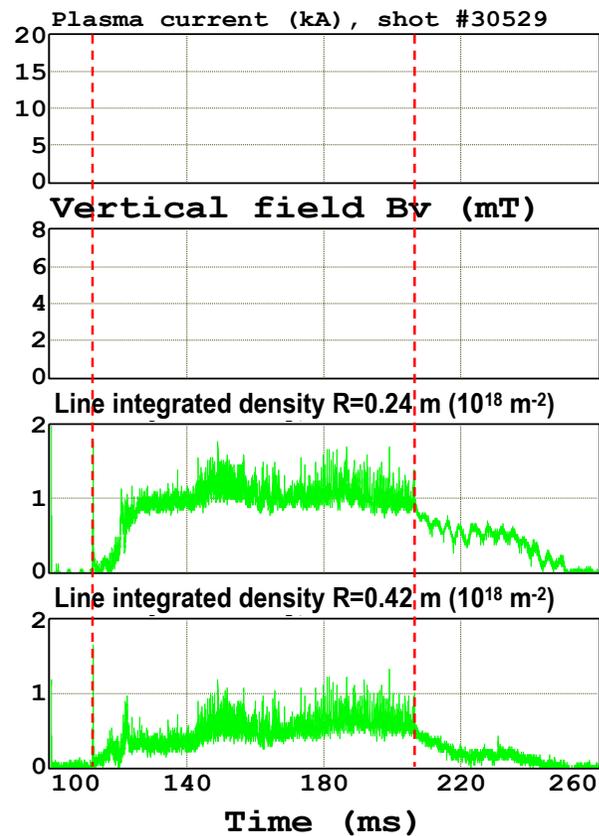


**OLD antennae with  
"poloidal" slow down:**

$$N_{tor} \approx (1.0 - 15)$$

$$N_{pol} \approx (7 - 8)$$

# Total driven current comprises at least by 80% of noninductively driven current by LH waves at 900 MHz and 60 kW RF power launched



**Contribution of the inductive PF coil flux doesn't exceed 20%**

- Inductive flux conserved in the plasma current

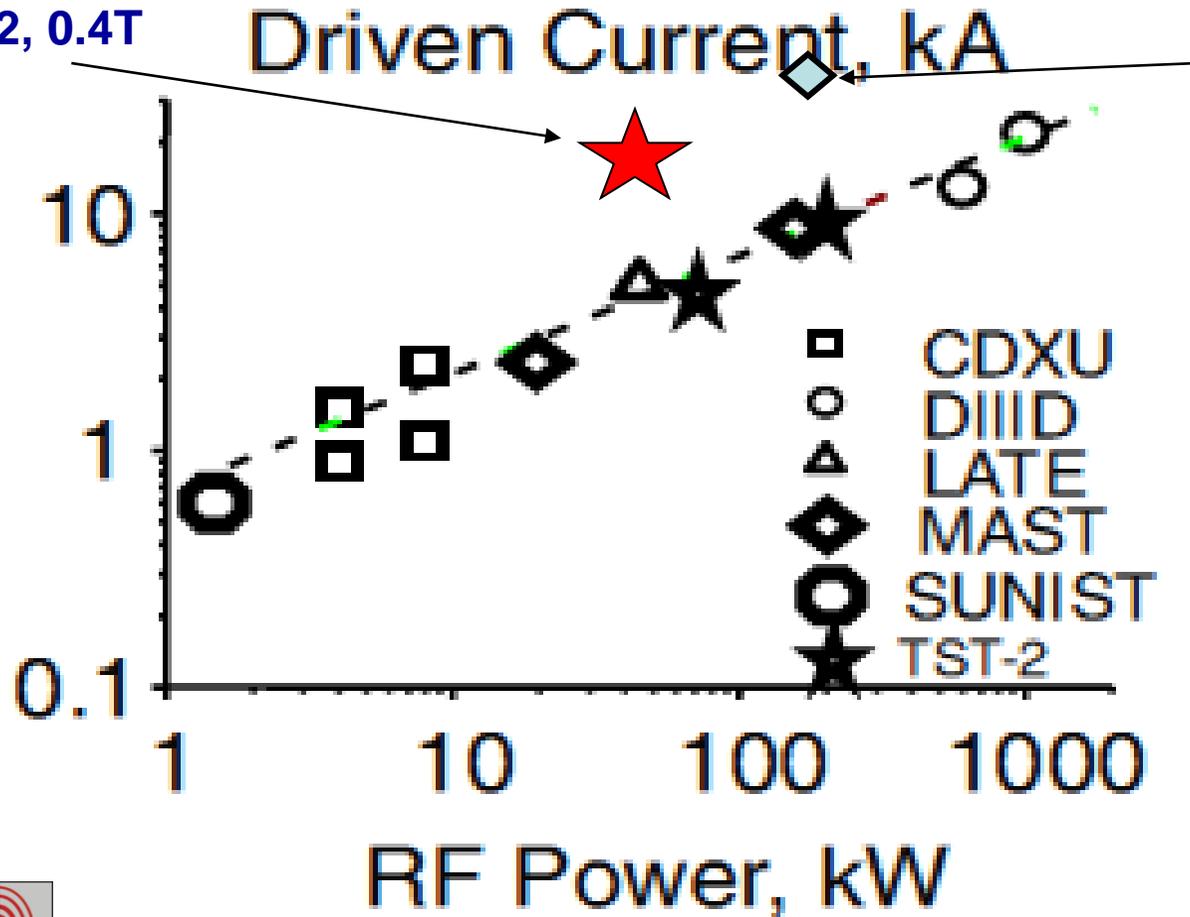
$$\Psi_{\text{surf}} = I_p L_{\text{ext}} = 7,1 \text{ mWb}$$

- Vertical field ramp-up flux

$$\Psi_{\text{PF}} = \Delta t_{\text{ramp-up}} U_{\text{surf}} = 1,5 \text{ mWb}$$

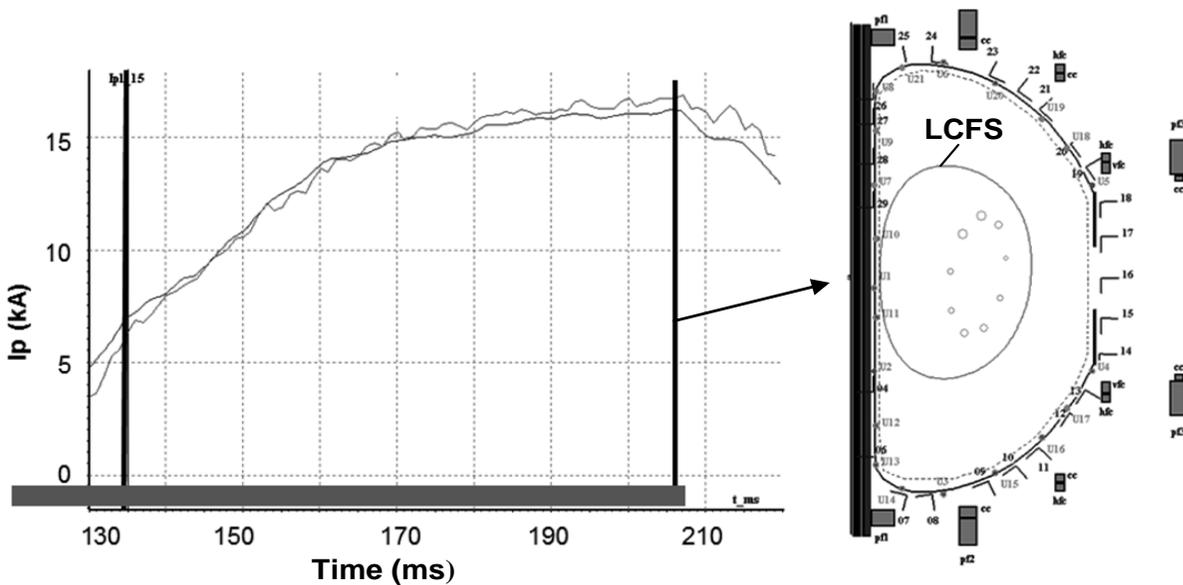
# Comparison with other experiments...

Globus-M, 2011-12, 0.4T



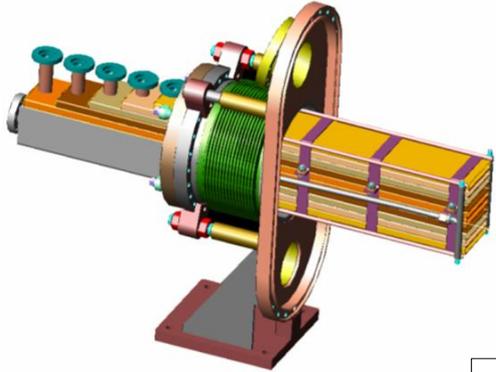
MAST, 2009

# Noninductively driven current forms plasma column with closed magnetic surfaces

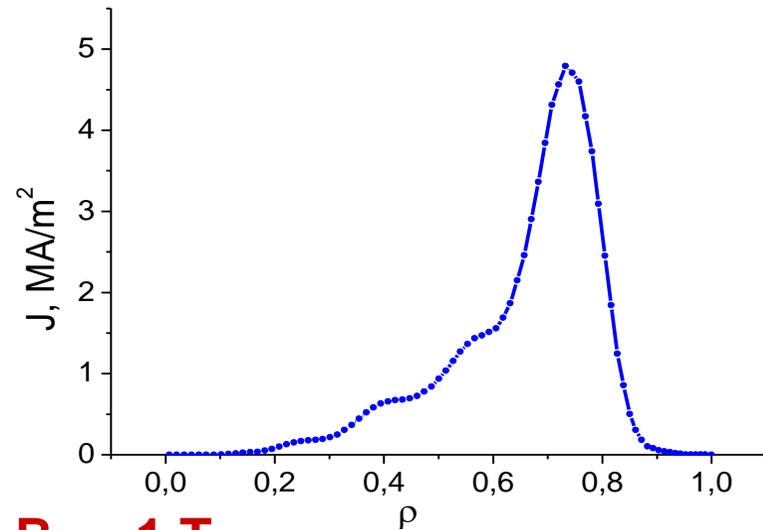
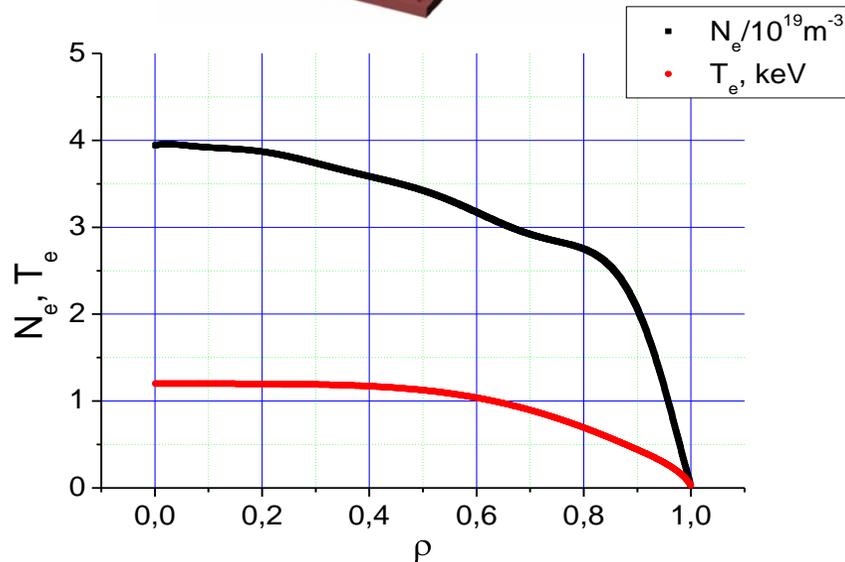


Magnetic reconstruction by current filament method was used

# Preliminary ray tracing simulations of noninductive CD drive by LH waves in Globus-M2



10-waveguides Grill may be oriented both in poloidal and toroidal directions



$B_T = 1 \text{ T}$

$I_P(\text{total}) = 0.5 \text{ MA};$

$I_P(\text{LHCD}) = 0.32 \text{ MA}.$

$F = 2.45 \text{ GHz};$

$P_{\text{LH}} = 0.5 \text{ MW}$

# Globus-M2 parameter modeling

# Globus-M regimes provides basis for ASTRA code modeling of Globus-M2 parameters

Instruments: ASTRA, NCLASS, NUBEAM codes

Reference case: Globus-M high density regime with  $\chi_e \approx 8 \text{ m}^2/\text{s}$

Electrons heat transport

$$\tau_E^{IPB98(2)} \sim I_p^{0.93} \cdot B^{0.15}$$

$$\chi_e^{Gl-M2} \approx \frac{1}{2.7} \chi_e^{Gl-M} \approx 3 \text{ m}^2 / \text{s}$$

Spherical tokamak scaling (M.Valovic et al, Nucl. Fusion, 2009, V49, p075016)

$$\tau_E^{Valovic} \sim I_p^{0.59} \cdot B^{1.4}$$

$$\chi_e^{Gl-M2} \approx \frac{1}{6.2} \chi_e^{Gl-M} \approx 1.3 \text{ m}^2 / \text{s}$$

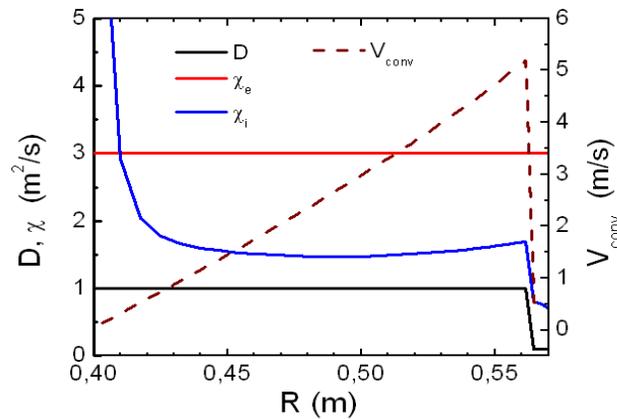
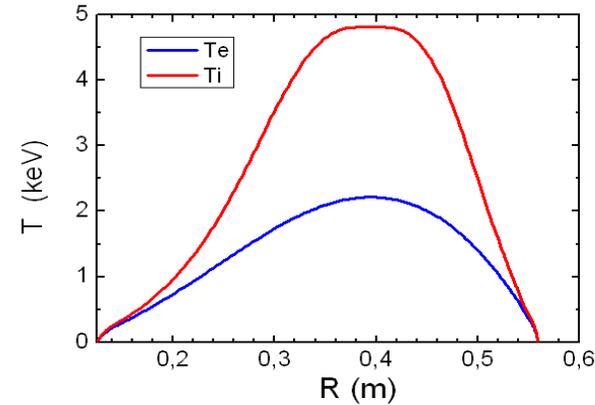
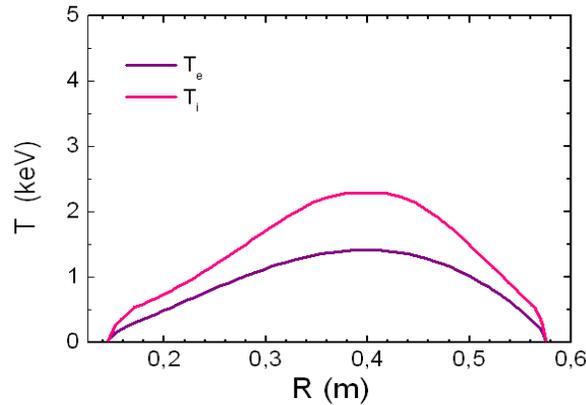
Ions heat transport

$$\chi_i = \chi_i^{NEOCL} + \chi_i^{AN}$$

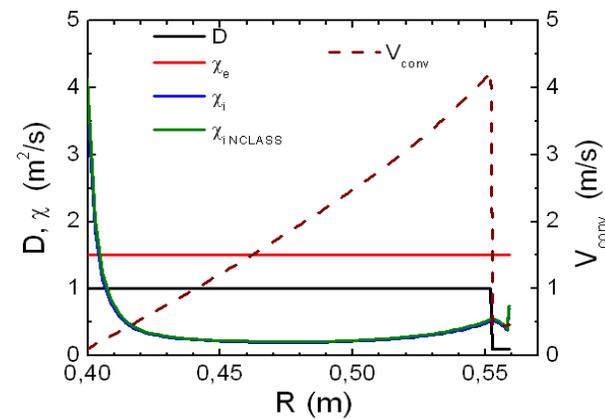
$$\chi_i^{AN} \approx D$$

Particle transport – unchanged

# ASTRA simulation of $E_D = 30$ keV, $P_{NB} = 1$ MW NB injection into Globus-M2 discharge demonstrates high value of temperature at density $\sim 0.7 \cdot 10^{20} \text{m}^{-3}$



**IPB98(y,2)**



**Valovic scaling**

# Conclusions

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**The analysis of Globus-M results on thermal and fast particle confinement, RF heating and CD led to the conclusion on magnetic field increase in Globus-M2**

**Upgrade allows for retaining the vacuum vessel and most part of the diagnostics and heating systems.**

**Substantial improvement in parameters can be expected for Globus-M2: density up to  $10^{20} \text{ m}^{-3}$  and temperatures in keV range could be attained with high confidence.**

**The dimensionless parameter range for Globus-M2 allow conditions characteristic for a compact fusion neutron source experimental support.**